



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 93 TO

FACILITY OPERATING LICENSE NPF-5

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

DOCKET NO. 50-366

1.0 INTRODUCTION

By letter dated February 9, 1988, Georgia Power Company (the licensee) requested a change to the Technical Specifications (TS) for Hatch Unit 2. The change would add a footnote to TS Table 3.3.2-3 to explain that the Reactor Water Cleanup (RWCU) system high differential flow isolation signal includes a 45-second time delay.

2.0 EVALUATION

Table 3.3.2-3 indicates a valve isolation response time of 13 seconds or less for the RWCU system upon receipt of a high differential flow signal. However, not stated is the fact that the isolation signal is not generated until the high differential flow condition has been present for 45 seconds. This delay is built in to avoid spurious signals that could be generated during startup and shutdown, instrument testing, pump start, or demineralizer change out.

The time delay has existed since Hatch Unit 2 was built and is a common design feature of BWRs. The BWR Standard Technical Specifications (NUREG-0123, Revision 3) acknowledge the existence of the delay timer:

Containment isolation for RWCU pipe breaks is provided by one of the following Class 1E circuits: Reactor Vessel Water level Low Low, RWCU Area Temperature High, or RWCU Area Ventilation Differential Temperature High. The RWCU high differential flow isolation is a single channel system provided for leak-before-break protection, but it is not relied upon in the accident analysis.

The change proposed by the licensee is thus a clarification of the design as actually exists at the plant. The change will not affect plant operation or plant safety in any manner. On this basis, the staff finds the proposed change acceptable.

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### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register on May 4, 1988 (53 FR 15911), and consulted with the state of Georgia. No public comments were received, and the state of Georgia did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Lawrence P. Crocker, PDII-3/DRP-I/II

Dated: June 9, 1988

Sholly Coordinator

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No significant hazards consideration comments received: No.

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Lawrence P. Crocker, Project Manager  
Project Directorate II-3  
Division of Reactor Projects-I/II

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Hatch Reading

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