

TABLE 4.3-1

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	ANALOG CHANNEL OPERATIONAL TEST	TRIP ACTUATING DEVICE OPERATIONAL TEST	ACTUATION LOGIC TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
1. Manual Reactor Trip	N.A.	N.A.	N.A.	R	N.A.	1, 2, 3*, 4*, 5*
2. Power Range, Neutron Flux						
a. High Setpoint	S	D(2, 4), M(3, 4, 14) Q(4, 6), R(4, 5a)#	Q	N.A.	N.A.	1, 2
b. Low Setpoint	S	R(4)#	Q	N.A.	N.A.	1###, 2
3. Power Range, Neutron Flux, High Positive Rate	N.A.	R(4)#	Q	N.A.	N.A.	1, 2
4. Power Range, Neutron Flux, High Negative Rate	N.A.	R(4)#	Q	N.A.	N.A.	1, 2
5. Intermediate Range, Neutron Flux	S	R(4, 5a)#	Q	N.A.	N.A.	1###, 2
6. Source Range, Neutron Flux	S	R(4, 5b, 12)#	Q(9)	N.A.	N.A.	2##, 3, 4, 5
7. Overtemperature ΔT	S	R(13)#	Q	N.A.	N.A.	1, 2
8. Overpower ΔT	S	R#	Q	N.A.	N.A.	1, 2
9. Pressurizer Pressure-Low (Above P-7)	S	R#	Q**	N.A.	N.A.	1
10. Pressurizer Pressure-High	S	R#	Q	N.A.	N.A.	1, 2
11. Pressurizer Water Level-High (Above P-7)	S	R#	Q	N.A.	N.A.	1

BRAIDWOOD - UNITS 1 & 2
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AMENDMENT NO. 2

TABLE 4.3-1 (Continued)

TABLE NOTATIONS

(12) At least once per 18 months during shutdown verify that on a simulated Boron Dilution Doubling test signal CVCS valves 112D and E open and 112B and C close within 30 seconds.

(13) CHANNEL CALIBRATION shall include the RTD bypass loops flow rate.

(14) On a one time basis through June 15, 1988, the periodicity of this surveillance for Unit 1 only may be extended from 31 days (monthly) to 41 days.

ATTACHMENT B

SIGNIFICANT HAZARDS CONSIDERATIONS

Commonwealth Edison has evaluated this proposed amendment and determined that it involves no significant hazards considerations. According to 10 CFR 50.92(c), a proposed amendment to an operating license involves no significant hazards considerations if operation of the facility in accordance with the proposed amendment would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

This proposed amendment requests a one-time only change (effective until June 15, 1988) for Unit 1, to Technical Specification 4.3.1.1 for the Reactor Trip System Instrumentation. The change proposes, on a one time basis, to extend the monthly surveillance periodicity for the Power Range Neutron Flux High Setpoint from 31 days to 41 days.

The probability or consequences of accidents while the incore moveable detector system is degraded are not increased because extension of a surveillance interval does not significantly affect the probability for the occurrence of an accident. Shiftly and 18-month surveillances are performed on the equipment to demonstrate operability. Given that the equipment will still function, the consequences of an accident remain unaffected.

The possibility of a new or different kind of accident from any accident previously evaluated is not created because the ability of the moveable detector system to measure core power distributions is not significantly degraded. Also, the impact on the ability to measure quadrant tilt and core average axial power shape is negligible. Finally, no new equipment is introduced and no equipment is operated in a new or different manner.

The margin of safety is not reduced because sufficient thimbles will be available to ensure that no quadrant will be unmonitored. The plugged tubes that necessitate this change are not concentrated in any quadrant or core region. A full core flux map completed on May 27, 1988 utilizing 42 thimbles indicated acceptable comparison of Incore Axial Flux Difference.

Based upon the above, Commonwealth Edison believes this amendment involves no significant hazards consideration.

ATTACHMENT C

SAFETY EVALUATION

The Movable Incore Detector System (MIDS) consists of 58 incore flux thimbles to permit measurement of the axial neutron flux distribution within the reactor core. Six movable neutron flux detectors are available to scan the length of the 58 selected fuel assemblies to provide remote reading of the axial flux distribution. The MIDS is used for confirmatory information and is not required for the day to day safe operation of the core. Daily core power performance is monitored by the excore detectors.

Commonwealth Edison believes there is no safety significance in extending the monthly surveillance interval for the Power Range Neutron Flux High Setpoint for ten (10) additional days, while the plant operates at 75% power. There are other functional tests that are performed on a shiftly and 18 month basis which would detect degradation or inoperability of a channel or component so appropriate actions can be initiated. These other types of testing include low and high voltage power supply adjustments, detector current and full power meter checks, a flux difference amplifier check, and power range rate circuit and bistable relay driver adjustments. Performance of these ensures that the affected channels operate within specified limits.

In addition, the channel checks perform a qualitative assessment of channel behavior. If one channel's indication deviates from its other associated channels, this can be readily detected and corrective actions would be taken.

Commonwealth Edison believes that the routine shiftly and 18 month surveillances that will continue to be performed during the extended 10 day period will detect instrument failure mechanisms. Therefore, any failures can be corrected so the Unit will not operate in a degraded condition. Also, the probability of an accident occurring during this surveillance extension is minimal. Typically there are redundant trip signals to respond to an accident scenario. It is highly unlikely these trip signals will not function as a result of this surveillance extension. Thus, Commonwealth Edison believes this on-time extension request should not significantly impact the safety of Braidwood Station.