

09 APR 1986

Docket No. 50-289

MEMORANDUM FOR: Harry Kister, Chief
Projects Branch No. 1

FROM: Allen R. Blough, Chief
Reactor Projects
Section 1A

SUBJECT: TMI-1 STATUS REPORT FOR THE PERIOD MARCH 28 - April 4, 1986

Enclosed is the TMI-1 weekly status report from the NRC Resident Office for the subject period. Cold shutdown work activities continued. The NRC staff continued to follow the outage status, including the eddy current testing of steam generator tubes, and to conduct random inspection of the test and work activities.

TMI-1 status reports are intended to provide NRC management and the public with highlights from an NRC regulatory perspective of TMI-1 activities for the previous week. Subsequent inspection reports will address many of these topics in more detail.



Allen R. Blough, Chief
Reactor Projects Section 1A
Branch 1, DRP

Enclosure:
As stated

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TMI1 WEEKLY STATUS ITEMS - 0001.0.0
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09 APR 1986

cc w/enclosure:

- F. Miraglia, NRR
- W. Travers, NRR
- J. Thoma, NRR
- J. Partlow, IE
- T. Gerusky, BRP/DER, Commonwealth of Pennsylvania
- R. Benko, Governor's Office of Policy, Commonwealth of Pennsylvania
- TMI Alert
- Susquehanna Valley Alliance
- Friends & Family of TMI
- Public Document Room
- Local Public Document Room

bcc w/enclosure:

- K. Abraham, RI
- P. Lohaus, RI
- R. Starostecki, RI
- H. Kister, RI
- R. Conte, RI (20 cys)
- W. Baunack, RI
- Region I Docket Room (w/concurrences)

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for RI:DRP
Conte
4/9/86

RI:DRP
Blough
4/9/86

RI:DRP
Kister
4/9/86

ENCLOSURE

TMI-1 STATUS REPORT FOR THE PERIOD March 28 - April 4, 1986

1. Plant Status

As of 8:00 a.m. on April 4, 1986, TMI-1 was in a cold shutdown condition with reactor vessel water level maintained at the reactor vessel nozzles and the primary side of the steam generators drained for eddy current testing (ECT). The decay heat system was maintaining reactor vessel water temperature at approximate 80-100 F.

2. Facility Operations Summary

Cold shutdown activities continued throughout the period. In addition to eddy current testing, those activities included primarily preventive/ corrective maintenance work and modification work in order to make progress on those items that must be completed before the Cycle 6 refueling outage startup. Testing continued on safety related motor-operated valves and containment isolation valves.

3. Special Interest Items

Eddy Current Testing of Steam Generator Tubes

The licensee made progress during the week in obtaining eddy current data and in conducting analysis of that data. Licensee and contractor personnel collected and computerized data on site, and analyzed that data at a location off site. The initial tube sample size was approximately 1500 tubes for each of the two steam generators. For the "B" steam generator, the licensee has nearly completed analysis of data. Thusfar, defects greater than 40% throughwall have been identified in three tubes.

Data collection and analysis continues for the "A" steam generator.

Radiological Release During Reactor Building Purging, March 25 - April 7, 1986

Last week's status report provided radiological release information for controlled reactor building purging up to March 24, 1986. For the period March 25 to April 7, 1986, the licensee reported the release of 194 Ci of noble gases. Also, released were approximately 3.156 E-5 Ci Particulate, Tritium and Iodine, predominantly Iodine 131. This converts to an integrated dose at the site boundary of 4.94 E-3 mRad gamma and 2.16 E-2 mRad beta, which is within technical specification limits.

4. NRC Staff Status During the Period

During this report period, the NRC staff on site consisted of the senior resident inspector and a resident inspector, supplemented by radiation specialists from Region I.

The resident staff continued to follow plant status and outage work and conducted random inspections of these activities. The two radiation specialists from Region I were on site to review outage radiological control practices and, also, to follow-up on the radiological events on March 24-25, 1986.

5. Systematic Assessment of Licensee Performance (SALP)

The NRC Region I SALP board met on January 24, 1986, to discuss licensee performance during the TMI-1 restart period of September 16, 1985, to January 10, 1986. The NRC and GPUN managements met to discuss the SALP at 10:00 a.m., March 31, 1986, at the NRC Region I Office in King of Prussia. No members of the public attended this meeting.

At 7:30 p.m. on Thursday, April 24, 1986, members of NRC Region I management will be available at the NRC's Middletown office (Downtown Mall, 100 Brown Street, Middletown, Pennsylvania) to answer questions from the public regarding both the SALP and NRC inspection programs at TMI-1. Because of space limitations at the Middletown Office, it is important for those planning to attend the public meeting to contact Allen R. Blough at (215) 337-5146 or (717) 948-1160. If the number of respondents is large, an alternate location in the Middletown-Harrisburg area will be arranged. Public response to this planned meeting has been minimal thusfar.

6. NRC (TMI-1) Staff Composition During Period

The NRC staff at TMI-1 was comprised of the following personnel during the period:

- R. Conte, Senior Resident Inspector
- D. LaQuia, Project Engineer
- A. Weadock, Reactor Engineer
- F. Young, Resident Inspector
- C. Hix, Secretary