



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO.63 TO

FACILITY OPERATING LICENSE NO. NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

NORTH ANNA POWER STATION, UNIT NO. 2

DOCKET NO. 50-339

Introduction:

By letter dated February 6, 1986, the Virginia Electric and Power Company (the licensee) proposed changes to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). Specifically, the changes pertain to Section 3/4.6.3, Table 3.6-1, "Containment Isolation Valves," for the NA-1&2 TS. The changes reflect a design modification for the installation of a new containment isolation valve in the letdown line of NA-1&2.

Discussion:

The existing three parallel valves (HCV-1200A, HCV-1200B and HCV-1200C for NA-1 and HCV-2200A, HCV-2200B, and HCV-2200C for NA-2) in the letdown line were designated as containment isolation valves inside containment. These valves are located at an elevation that is below the maximum flood level in the containment and, therefore, do not meet the guidelines of Regulatory Guide 1.97 concerning environmental qualification. Moreover, these valves have experienced leakage problems relative to meeting the leak testing requirements of Appendix J to 10 CFR Part 50.

The licensee's proposed resolution is to add a new valve in the letdown line of each unit that will serve as the containment isolation valve inside containment. The new valve will be located above the maximum flood level and will meet the design criteria of containment isolation valves. Also, this single valve will provide greater assurance of leakage integrity of the line. The existing outboard containment isolation valve will remain unchanged.

The staff finds this design modification acceptable and to be in compliance with Standard Review Plan Section 6.2.4, "Containment Isolation System," and 6.2.6, "Containment Leakage Testing."

In conjunction with the above design modification, the licensee has proposed to change the TS Table 3.6-1, "Containment Isolation Valves," to reflect the change. The containment isolation function of the valves (TV 1204A for

NA-1 and TV 2204A for NA-2) negates this function previously assigned to the following valves in the letdown line: the three parallel valves identified above, a relief valve (RV-1203 for NA-1 and RV-2203 for NA-2), and a manual valve (HCV-1142 for NA-1 and HCV-2142 for NA-2). The relief valve discharges to the pressurizer relief tank, and the manual valve is in a line which connects the letdown line to the Reactor Heat Removal heat exchanger. The existing valves will not be physically removed but will no longer provide a containment isolation function.

Evaluation:

Based on the above, the staff finds the proposed changes to appropriately meet the provisions for the containment isolation letdown line and, therefore, finds the change to be acceptable.

Finally, while this Safety Evaluation (SE) is applicable for both NA-1&2, the instant amendment supported by this SE is applicable at this time for only NA-2. The installation of the new containment isolation valve for NA-1 will not be installed until the forthcoming NA-1 refueling outage which is approximately 15 months hence. We will issue a similar amendment for NA-1 at that time.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 4, 1986

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