



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated February 6, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

B.C. Buckley for

Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 4, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 63

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.1.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

- a. Verifying that on a Phase A containment isolation test signal, each Phase A isolation valve actuates to its isolation position.
- b. Verifying that on a Phase B containment isolation test signal, each Phase B isolation valve actuates to its isolation position.
- c. Verifying that on a Containment Purge and Exhaust isolation signal, each Purge and Exhaust valve actuates to its isolation position.
- d. Cycling each weight or spring loaded check valve not testable during plant operation, through one complete cycle of full travel and verifying that each check valve remains closed when the differential pressure in the direction of flow is less than 1.2 psid and opens when the differential pressure in the direction of flow is greater than or equal to 1.2 psid but less than or equal to 5.0 psid.

4.6.3.1.3 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

TABLE 3.6-1

CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC.)</u>
A. PHASE "A" ISOLATION		
1. MOV-2380	Reactor Coolant Pump Seal Water Return	10
2. MOV-2381	Reactor Coolant Pump Seal Water Return	10
3. Deleted		
4. Deleted		
5. TV-2204A	Reactor Coolant Letdown Line	10
6. TV-2204B	Reactor Coolant Letdown Line	10
7. TV-S1200	Nitrogen to Pressurizer Relief Tank and SI Accumulators	60
8. TV-DG200A	Primary Drains Transfer Tank Pump Discharge	60
9. TV-DG200B	Primary Drains Transfer Tank Pump Discharge	60
10. TV-DA200A	Containment Sump Pump Discharge to Waste Drain Tanks	60
11. TV-DA200B	Containment Sump Pump Discharge to Waste Drain Tanks	60
12. TV-BD200A	Steam Generator Blowdown	60
13. TV-BD200B	Steam Generator Blowdown	60
14. TV-BD200C	Steam Generator Blowdown	60

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC.)</u>
31. 2-WT-447*	Steam Generator Wet Layup	NA
32. 2-WT-448*	Steam Generator Wet Layup	NA
33. 2-SI-83*	High Head Safety Injection, (Boron Injection Tank Bypass)	NA
34. NA*	Fire Protection Supply (Penetration 34)	NA
E. REMOTE MANUAL		
1. MOV-QS201A*	Quench Spray Pump Discharge	NA
2. MOV-QS201B*	Quench Spray Pump Discharge	NA
3. MOV-RS255A#*	Recirc. Spray Pump Suction	NA
4. MOV-RS255B#*	Recirc. Spray Pump Suction	NA
5. MOV-2860A#*	LHSI Pump Suction From Containment Sump	NA
6. MOV-2860B#*	LHSI Pump Suction From Containment Sump	NA
7. MOV-RS256A*	Recirculation Spray Pump Discharge	NA
8. MOV-RS256B*	Recirculation Spray Pump Discharge	NA
9. MOV-SW203A*	Service Water to Recirculation Spray Coolers	NA
10. MOV-SW203B*	Service Water to Recirculation Spray Coolers	NA
11. MOV-SW203C*	Service Water to Recirculation Spray Coolers	NA
12. MOV-SW203D*	Service Water to Recirculation Spray Coolers	NA
13. MOV-SW204A*	Service Water from Recirculation Spray Coolers	NA

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Amendment No. 25, 63

TABLE 3.6-1 (Con't.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
14. MOV-SW204B*	Service Water from Recirculation Spray Coolers	NA
15. MOV-SW204C*	Service Water from Recirculation Spray Coolers	NA
16. MOV-SW204D*	Service Water from Recirculation Spray Coolers	NA
17. TV-CV200*	Containment Air Ejector Suction	NA
18. MOV-2869A*	High Head Safety Injection to RCS Except Boron Injection Line	NA
19. MOV-2836*	High Head Safety Injection to RCS Except Boron Injection Line	NA
20. MOV-2869B*	High Head Safety Injection to RCS Except Boron Injection Line	NA
21. Deleted		
22. Deleted		
23. Deleted		
24. MOV-2890A*	LHSI Pump Discharge to Reactor Coolant System Hot Legs	NA
25. MOV-2890B*	LHSI Pump Discharge to Reactor Coolant System Hot Legs	NA
26. MOV-2890C*	LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC.)</u>
37. 2-FW-94#	Feedwater to Steam Generators	NA
38. 2-FW-126#	Feedwater to Steam Generators	NA
39. 2-WT-41#	Chemical Feed Lines	NA
40. 2-WT-53#	Chemical Feed Lines	NA
41. 2-WT-69#	Chemical Feed Lines	NA
42. 2-FW-70#	Auxiliary Feedwater to Steam Generator	NA
43. 2-FW-102#	Auxiliary Feedwater to Steam Generator	NA
44. 2-FW-134#	Auxiliary Feedwater to Steam Generator	NA
45. 2-RS-103#	Casing Cooling to Outside Recirculation Spray Pump	NA
46. 2-RS-118#	Casing Cooling to Outside Recirculation Spray Pump	NA
47. NA	Fire Protection Supply (Penetration 34)	NA

TABLE 3.6-1 (Con't.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
G. STEAM LINE ISOLATION		
1. TV-MS201A#	Main Steam Line Trip Valve	5
2. TV-MS201B#	Main Steam Line Trip Valve	5
3. TV-MS201C#	Main Steam Line Trip Valve	5
H. RELIEF		
None		

Valve not subject to Type "C" leakage test

* Valve position maintained by administrative control

NA - Not applicable

** - Requires testing per Technical Specifications 4.6.3.1.1a. or 4.6.3.1.2d.