

## UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 29, 1998

Mr. D. N. Morey
Vice President - Farley Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

SUBJECT

RELIEF REQUEST FOR THE PUMP AND VALVE INSERVICE TESTING PROGRAM - JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

(TAC NOS. M99186 AND M99187)

Dear Mr. Morey:

This letter provides the results of the U.S. Nuclear Regulatory Commission (NRC) staff's review of the Joseph M. Farley Nuclear Plant (FNP), Unit 1 third 10-year interval inservice testing (IST) program for pumps and valves. The third interval covers the period from December 1, 1997, to November 30, 2007. For FNP Unit 2, the IST program covers the second 10-year interval from December 1, 1997, to July 30, 2001, and a portion of the third 10-year interval from August 1, 2001, to November 30, 2007. In a letter dated March 20, 1997, NRC approved the use of the American Society of Mechanical Engineers (ASME) Operations and Maintenance Committee of ASME (OM) Code-1990, Subsection ISTB for Pumps and Subsection ISTC for Valves to udpate the IST programs for FNP Units 1 and 2.

By letters dated July 3, 1997, and January 22, 1998, Southern Nuclear Operating Company, Inc. (SNC) submitted its third and second 120-month updates to its IST program (for the interval from December 1, 1997, through November 30, 2007) of pumps and valves for FNP, Units 1 and 2, respectively. In the updated program, SNC submitted five pump relief requests for both Units 1 and 2, five valve relief requests for Unit 1, and seven valve relief requests for Unit 2. The staff and SNC discussed these relief requests on November 18, 1997, and April 28, 1998. In response to the staff's questions and comments, SNC provided additional information by letter dated January 22, 1998, and indicated that more information would be provided at a later date. In a subsequent letter dated April 6, 1998, SNC withdrew certain relief requests, and revised certain other relief requests. However, in its April 6, 1998, letter, SNC submitted three new valve relief requests for both Units 1 and 2. After review of these additional relief requests, a conference call was held on April 28, 1998, to discuss the NRC's concerns and comments. In a letter dated August 7, 1998, SNC withdrew one of the three new relief requests and revised the other two relief requests.

The staff has completed its review of all relief requests submitted in conjunction with the additional information provided in the previously discussed letters. The enclosed Safety Evaluation provides the staff's evaluation and conclusions. With the exception of relief requests Q1P16-RR-V-3 and Q2P16-RR-V-3, the proposed alternatives described in the other remaining relief requests are authorized pursuant to 10 CFR 50.55a(a)(3)(ii), on the basis that the SNC's proposed alternatives provide reasonable assurance of operational readiness for the affected pumps and valves and that imposition of the Code requirements would result in hardship

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without a compensating increase in the level of quality and safety. With respect to relief requests Q1P16-RR-V-3 and Q2P16-RR-V-3, the use of grouping and sampling approach for Valve QV661 is denied, because it does not meet the size requirement of Generic Letter 89-04, Position 2. SNC should remove Valve QV661 from these relief requests.

The changes in the scope of the IST program, cold shutdown justifications, refueling outage justifications, and IST program commitments have not been reviewed in detail and are subject to NRC inspection.

If you have any questions regarding this response to your requests, please have your representative contact Jacob I. Zimmerman at (301) 415-2426.

Sincerely,

Herbert N. Berkow, Director

Project Directorate II-2

Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosure: Safety Evaluation

cc w/encl: See next page

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Sincerely.

ORIGINAL SIGNED BY D. JAFFE FOR:

Herbert N. Berkow, Director Project Directorate II-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosure: Safety Evaluation

cc w/encl: See next page

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