

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Millstone Nuclear Power Station Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 4 2 3 1	PAGE(S) OF 0 4
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TITLE (4)
Area Temperature Monitoring - MS-01

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 9	0 2	8 6	8 6	0 5 0	0 3	0 9	3 0	8 8			0 5 0 0 0
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11):											

OPERATING MODE (9) 1	POWER LEVEL (10) 1 0 0	20.402(b) <input type="checkbox"/> 20.406(a)(1)(ii) <input type="checkbox"/> 20.406(a)(1)(iii) <input type="checkbox"/> 20.406(a)(1)(iv) <input type="checkbox"/> 20.406(a)(1)(v) <input type="checkbox"/> 20.406(a)(1)(vi)	20.406(a) <input type="checkbox"/> 20.38(a)(1) <input type="checkbox"/> 20.38(a)(2) <input type="checkbox"/> 20.73(a)(2)(i) <input type="checkbox"/> 20.73(a)(2)(ii) <input type="checkbox"/> 20.73(a)(2)(iii)	20.73(a)(2)(iv) <input type="checkbox"/> 20.73(a)(2)(v) <input type="checkbox"/> 20.73(a)(2)(vi) <input type="checkbox"/> 20.73(a)(2)(vii) <input type="checkbox"/> 20.73(a)(2)(viii)	20.73(b) <input type="checkbox"/> 20.73(c) <input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 306A) Special Report
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LICENSEE CONTACT FOR THIS LER (12)

NAME Vere R. Joseph - Associate Engineer, Extension 5571	TELEPHONE NUMBER AREA CODE: 2 0 3 4 4 7 - 1 7 9 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (if yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH DAY YEAR
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ABSTRACT (Limit to 1400 words) (16) - Additionally, attach a separate submission (17)

This Special Report revision is being submitted to provide an updated status to Special Report LER 86-050-72 and pursuant to Plant Technical Specifications 3.7.14b and 6.9.2 to report area temperature excursions in the Main Steam Valve Building. Plant Technical Specification 3.7.14b requires that a Special Report be submitted to the NRC if one or more areas exceed the specified temperature limit by less than 20 degrees Fahrenheit for more than 8 hours. LER 86-050-02 reported that Main Steam Valve Building area MS-01 had exceeded the 120 degree Fahrenheit specified limit on 12 occasions. Area temperature element 3ECS-TE119, located on the top floor elevation 71'-2" is the element within the building which exceeded the limits.

All environmentally qualified equipment was verified to be operable. Due to the number of temperature excursions in this area, a permanent modification was installed. The modification was tested and verified operable during the summer of 1988. There have been no other temperature excursions to date.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED CMB NO. 3150-0104
EXPIRES 8/31/86

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TEXT (If more space is required, use additional NRC Form 388A's) (17)

This Special Report revision is being submitted to provide an updated status to Special Report LER 86-050-02. Special Report LER 86-050-02 was submitted pursuant to Special Report LER 86-050-01 and Plant Technical Specifications 3.7.14b and 6.9.2 to report area temperature excursions in the Main Steam Valve Building. Plant Technical Specification, 3.7.14b requires that a Special Report be submitted to the NRC if one or more areas exceed the specified temperature limit by less than 20 degrees Fahrenheit for more than 8 hours. LER 86-050-02 reported that Main Steam Valve Building area MS-0) had exceeded the 120 degree Fahrenheit specified limit on twelve occasions. Area temperature element 3ECS-TE119, located on the top floor elevation 71'-2" is the only element within the building exceeding the limits.

At 0139 on September 2, 1986, 3ECS-TE119 reached a temperature of 124.5 degrees Fahrenheit. The plant entered the Action Statement at 0939. At the time that LER 86-050-01 was submitted (October 24, 1986), the temperature at elevation 71'-2" had been varying between 119 and 125 degrees Fahrenheit consistently since September 2, 1986 and there had been nine occurrences in which the 8 hour Action Statement had been entered. The dates of these excursions were September 2, 11, 15, 19, 24, 26, 29, and October 4 and 10 of 1986.

An analysis was performed for continued operability for a sustained temperature of 130 degrees Fahrenheit. The shortest thermal life under these conditions for environmentally qualified equipment is greater than five years.

Initially, temporary solutions were implemented with limited success. However, the temporary modifications by themselves were not successful in maintaining the area temperature below the Technical Specification limit for all operating conditions. Therefore, a permanent modification to the building's heating and ventilation system was performed.

This Special Report revision is being submitted to provide an updated status on the permanent heating and ventilation modification. A "spot cooling" design was installed which provides cooling ducts in the proximate vicinity of environmentally qualified equipment. The complete modification was installed during the first refueling outage during the winter of 1987-88. The system was functionally tested and operability was verified during the summer of 1988 to ensure the effectiveness of the modification in warm weather conditions.

Special Report LER 86-050-01 stated that the temperature excursions within the Main Steam Valve Building were being reported as a single occurrence and would not be reported individually as they occur. Special Report LER 86-050-02 documented three additional occurrences (beyond the nine previously reported) in which the temperature for 3ECS-TE119 (area MS-01) exceeded the Plant Technical Specification limit by less than 20 degrees Fahrenheit for more than 8 hours. The dates of these temperature excursions were July 24, August 16 and 18 of 1987. In all cases the excursions were investigated and determined to be related to the previously identified problem. There have been no temperature excursions subsequent to the submittal of Special Report LER 86-050-02 and the completion of permanent heating and ventilation modifications. The temperature monitoring through the summer of 1988 without temperature excursions provided positive indication that this problem has been resolved.

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FACILITY NAME (1) Millstone Nuclear Power Station Unit 3	DOCKET NUMBER (2) 05000423	LER NUMBER (3)			PAGE (2)	
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		86	050	03		

TEXT (if more space is required, use additional NRC Form 388A x/117)

This Special Report is being submitted pursuant to Plant Technical Specifications 3.7.14b and 6.9.2.

The following is a list of affected equipment in Area MS-01 within the zone covered by 3ECS-TE119:

ECS-TE119

- 3DTM*AOV29A - Main Steam Line Drain
- 3DTM*AOV29B - Main Steam Line Drain
- 3DTM*AOV29C - Main Steam Line Drain
- 3DTM*AOV29D - Main Steam Line Drain
- 3DTM*AOV61A - Main Steam Line Drain
- 3DTM*AOV61B - Main Steam Line Drain
- 3DTM*AOV61C - Main Steam Line Drain
- 3DTM*AOV61D - Main Steam Line Drain
- 3DTM*SOV29A - Main Steam Line Drain
- 3DTM*SOV29B - Main Steam Line Drain
- 3DTM*SOV29C - Main Steam Line Drain
- 3DTM*SOV29D - Main Steam Line Drain
- 3DTM*SOV61A - Main Steam Line Drain
- 3DTM*SOV61B - Main Steam Line Drain
- 3DTM*SOV61C - Main Steam Line Drain
- 3DTM*SOV61D - Main Steam Line Drain
- 3MSS*CTV27A - Main Steam Isolation Trip Valve
- 3MSS*CTV27B - Main Steam Isolation Trip Valve
- 3MSS*CTV27C - Main Steam Isolation Trip Valve
- 3MSS*CTV27D - Main Steam Isolation Trip Valve
- 3MSS*HV28A - Main Steam Isolation Bypass
- 3MSS*HV28B - Main Steam Isolation Bypass
- 3MSS*HV28C - Main Steam Isolation Bypass
- 3MSS*HV28D - Main Steam Isolation Bypass
- 3MSS*MOV18A - Main Steam Pressure Relieving Valve Isolation Valve
- 3MSS*MOV18B - Main Steam Pressure Relieving Valve Isolation Valve
- 3MSS*MOV18C - Main Steam Pressure Relieving Valve Isolation Valve
- 3MSS*MOV18D - Main Steam Pressure Relieving Valve Isolation Valve
- 3MSS*MOV74A - Main Steam Pressure Relieving Bypass Valve
- 3MSS*MOV74B - Main Steam Pressure Relieving Bypass Valve
- 3MSS*MOV74C - Main Steam Pressure Relieving Bypass Valve
- 3MSS*MOV74D - Main Steam Pressure Relieving Bypass Valve
- 3MSS*PV20A - Main Steam Pressure Relieving Valve
- 3MSS*PV20B - Main Steam Pressure Relieving Valve
- 3MSS*PV20C - Main Steam Pressure Relieving Valve
- 3MSS*PV20D - Main Steam Pressure Relieving Valve

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TEXT (if more space is required, use additional NRC Form 388A's) (17)

- 3MSS*RE75 - Steam Release Monitor
- 3MSS*RE76 - Steam Release Monitor
- 3MSS*RE77 - Steam Release Monitor
- 3MSS*RE78 - Steam Release Monitor
- 3MSS*SOV20A - Steam Pressure Relieving Valve
- 3MSS*SOV20B - Steam Pressure Relieving Valve
- 3MSS*SOV20C - Steam Pressure Relieving Valve
- 3MSS*SOV20D - Steam Pressure Relieving Valve
- 3MSS*SOV28A1 - Main Steam Isolation Bypass
- 3MSS*SOV28A2 - Main Steam Isolation Bypass
- 3MSS*SOV28B1 - Main Steam Isolation Bypass
- 3MSS*SOV28B2 - Main Steam Isolation Bypass
- 3MSS*SOV28C1 - Main Steam Isolation Bypass
- 3MSS*SOV28C2 - Main Steam Isolation Bypass
- 3MSS*SOV28D1 - Main Steam Isolation Bypass
- 3MSS*SOV28D2 - Main Steam Isolation Bypass
- 3MSS*SV27A1A - Main Steam Isolation Trip Valve
- 3MSS*SV27A1B - Main Steam Isolation Trip Valve
- 3MSS*SV27A2A - Main Steam Isolation Trip Valve
- 3MSS*SV27A2B - Main Steam Isolation Trip Valve
- 3MSS*SV27A3A - Main Steam Isolation Trip Valve
- 3MSS*SV27A3B - Main Steam Isolation Trip Valve
- 3MSS*SV27A4A - Main Steam Isolation Trip Valve
- 3MSS*SV27A4B - Main Steam Isolation Trip Valve
- 3MSS*SV27B1A - Main Steam Isolation Trip Valve
- 3MSS*SV27B1B - Main Steam Isolation Trip Valve
- 3MSS*SV27B2A - Main Steam Isolation Trip Valve
- 3MSS*SV27B2B - Main Steam Isolation Trip Valve
- 3MSS*SV27B3A - Main Steam Isolation Trip Valve
- 3MSS*SV27B3B - Main Steam Isolation Trip Valve
- 3MSS*SV27B4A - Main Steam Isolation Trip Valve
- 3MSS*SV27B4B - Main Steam Isolation Trip Valve
- 3MSS*SV27C1A - Main Steam Isolation Trip Valve
- 3MSS*SV27C1B - Main Steam Isolation Trip Valve
- 3MSS*SV27C2A - Main Steam Isolation Trip Valve
- 3MSS*SV27C2B - Main Steam Isolation Trip Valve
- 3MSS*SV27C3A - Main Steam Isolation Trip Valve
- 3MSS*SV27C3B - Main Steam Isolation Trip Valve
- 3MSS*SV27C4A - Main Steam Isolation Trip Valve
- 3MSS*SV27C4B - Main Steam Isolation Trip Valve
- 3MSS*SV27D1A - Main Steam Isolation Trip Valve
- 3MSS*SV27D1B - Main Steam Isolation Trip Valve
- 3MSS*SV27D2A - Main Steam Isolation Trip Valve
- 3MSS*SV27D2B - Main Steam Isolation Trip Valve
- 3MSS*SV27D3A - Main Steam Isolation Trip Valve
- 3MSS*SV27D3B - Main Steam Isolation Trip Valve
- 3MSS*SV27D4A - Main Steam Isolation Trip Valve
- 3MSS*SV27D4B - Main Steam Isolation Trip Valve

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

General Offices • Seiden Street, Berlin, Connecticut

P.O. BOX 270
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September 30, 1988
MP-12277

Re: Plant Technical Specifications
3.7.14b and 6.9.2

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. NPF-49
Docket No. 50-423
Licensee Event Report 50-423/86-050-03


Gentlemen:

This letter forwards Licensee Event Report 86-050-03, a revised Special Report required to be submitted by October 1, 1988 in accordance with Licensee Event Report 86-050-02 and pursuant to Plant Technical Specifications 3.7.14b and 6.9.2.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

FOR: Stephen E. Scace
Station Superintendent
Millstone Nuclear Power Station


BY: John S. Keenan
Unit 2 Superintendent
Millstone Nuclear Power Station

SES/VRJ:mo

Attachment: LER 86-050-03

cc: W. T. Russell, Region 1 Administrator
D. H. Jaffe, NRC Project Manager, Millstone Unit Nos. 2 and 3
W. J. Raymond, Senior Resident Inspector, Millstone Unit Nos. 1, 2 and 3

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