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June 2, 1988

Mr. T. E. Murley
U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC. 20555

Subject: Braidwood Station Unit 2
Preservice Inspection (PSI) Program
NRC Docket No. 50-457

References: (a) February 23, 1988 letter from S.C. Hunsader to T.E. Murley
(b) March 4, 1988 letter from K.A. Ainger to T.E. Murley
(c) NUREG-1002, Safety Evaluation Report Supplement #6 dated May, 1988

Dear Mr. Murley:

Reference (a) provided for the NRC staff review Relief Request 2NR15 Rev. 0. Along with the relief request, reference (a) provided an interim report on the analysis performed in accordance to ASME Section XI, paragraph IWB-3600 as described in the relief request. Reference (a) also committed to submit a final report which would contain a more detailed description of the methodology, calculations and results as well as a fatigue crack growth analysis based on flaw specific location. Reference (b) provided additional information to the NRC regarding the fatigue evaluation bases in the interim report and discussed the anticipated content of the final report. Reference (c) provided the NRC staff's evaluation of the submittals.

The purpose of this letter is to provide you with the final report, MT-SME-200, entitled "Evaluation of an Indication in the Braidwood Unit 2 Loop 1 Elbow to Valve Weld Region". This final report contains the fatigue crack growth analysis performed at the flaw-specific location and additional details on the analytical methodology utilized. This report demonstrates that the indication in the elbow-to-valve weld is acceptable to the requirements of ASME Section XI.

This is being submitted for NRC review and acceptance. Please address any questions concerning this matter to this office.

Very truly yours,

Steven C. Hunsader
Nuclear Licensing Administrator

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cc: S. Sands (NRR)
W. Forney (Region III)
Braidwood Resident Inspector

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