

COMBUSTION ENGINEERING

September 30, 1988
LD-88-106

Docket No. STN 50-470F
(Project No. 675)

Mr. Frank J. Miraglia
Associate Director for Projects
Office of Nuclear Reactor Regulation
Attn: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Combustion Engineering Standard Safety Analysis Report -
Design Certification, Submittal Group D: Amendments to
Chapters 2, 3, 4, 5, 6, 7, and 18

Dear Mr. Miraglia:

This letter transmits the next in Combustion Engineering's series of modifications to the Combustion Engineering Standard Safety Analysis Report - FSAR (CESSAR-F). Since the purpose of our modifications is to upgrade our current Final Design Approval (FDA-2) to a Design Certification, we refer to the revised CESSAR-F as CESSAR-DC. As you are aware, the enhanced and expanded design described in CESSAR-DC will be called the System 80+™ Standard Design.

The enclosure to this letter provides you with our draft revisions to sections of Chapters 2, 3, 4, 5, 6, 7 and 18. The attachment to this letter provides a brief summary of the enclosed revisions.

Revisions relative to CESSAR-F are identified by a bar in the right-hand margin. To distinguish the amendments to CESSAR-F which are intended to lead to Design Certification from the amendments made in the course of updating CESSAR-F, we are using a lettered amendment system for CESSAR-DC. This lettered amendment system was introduced in the submittal of Reference (A). The enclosed revisions to Chapters 2, 3, 4, 5, 6, 7, and 18 are identified as Amendment D.

We hope to formally submit a set of printed CESSAR-DC volumes to you in the near future. With that submittal, we intend to formally request opening of a separate docket for CESSAR-DC, as described in References

SS10110320 880930
PDR ADOCK 05000470
PDC

Power Systems
Combustion Engineering, Inc.

1000 Prospect Hill Road
Post Office Box 500
Windsor, Connecticut 06095-0500

(203) 688-1911
Telex: 99297

D032
1/1

Mr. F. J. Miraglia
September 30, 1988

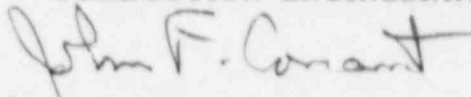
LD-88-106
Page 2

(B) and (C). The issue of the category and limit of the fees to which we will be liable [Reference (D)], however, continues to remain an impediment to our submittal. We are appreciative that this matter has been taken under advisement and is being given serious consideration [Reference (E)]. We are hopeful that, when this matter is resolved by the Commission, the result will be a satisfactory resolution of our concerns, removing this impediment to our submittal. In the meantime, however, we request that a means be found which will protect our rights and thus allow the technical review to proceed unencumbered.

If we can be of any additional assistance in this matter, please feel free to call me or Mr. E. H. Kennedy of my staff at (203) 285-4520.

Very truly yours,

COMBUSTION ENGINEERING, INC.


A. E. Scherer
Director
Nuclear Licensing

AES:lw

Attachment: As Stated
Enclosure: As Stated

cc: F. Ross (DOE - Germantown)
J. Devine (EPRI)

- References:
- (A) Letter, LD-88-026, A. E. Scherer (C-E) to F. J. Miraglia (NRC), dated April 11, 1988.
 - (B) Letter, LD-87-050, A. E. Scherer (C-E) to F. J. Miraglia (NRC), dated September 18, 1987.
 - (C) Letter, L. S. Rubenstein (NRC) to A. E. Scherer (C-E), dated October 13, 1987.
 - (D) Letter, LD-88-009, A. E. Scherer (C-E) to L. W. Zech, Jr. (NRC), dated February 1, 1988.
 - (E) Letter, V. Stello (NRC) to A. E. Scherer (C-E), dated June 3, 1988.

Summary of Significant Revisions for
CESSAR-DC, Amendment D*

Chapter 2 - Site Envelope Characteristics

Chapter 2 has been written to describe the System 80+TM standard site envelope.

Chapter 3 - Design of Structures, Components, Equipment & Systems

Primary changes to Chapter 3 cover modifications to the discussions of General Design Criteria, component classification, seismic qualification and environmental design. Future submittals will include the containment design and leak-before-break considerations.

Section 4.5 - Reactor Materials

Relatively minor changes were made to this section to update material specifications and to make editorial corrections.

Chapter 5 - Reactor Coolant System & Connected Systems

Primary changes to Chapter 5 include a description of the System 80+ reactor vessel, overpressure protection, and steam generator structural evaluations for the steam and feedwater line break events. The System 80+ pressurized thermal shock evaluation will be included in a future submittal.

* These revisions correspond to the scope of Chapters 6 and 10 of the EPRI ALWR Requirements Document.

Chapter 6 - Engineered Safety Features

Primary changes to Chapter 6 include a description of the System 80+TM Safety Depressurization System and the materials used in Engineered Safety Features systems. Containment spray and fission product removal systems will be described in a future submittal.

Chapter 7 - Instrumentation and Controls

Nuplex 80+ systems required for display, control and safe shutdown are described. The reactor protective system and engineered safety features actuation system will be described in a future submittal.

Chapter 18 - Human Factors Engineering

The Nuplex 80+ design approach has been described, concerning the role of human factors engineering.