



ARKANSAS POWER & LIGHT COMPANY

June 3, 1988

2CAN068801

Dr. Thomas E. Murley
Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

SUBJECT: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Reactor Containment Building
Integrated Leak Rate Test - May 1985
Revision 1

Dear Dr. Murley:

Attached is Revision 1 to the Reactor Containment Building Integrated Leak Rate Test Report for Arkansas Nuclear One, Unit 2, conducted May 1, 1985. The original report was submitted in accordance with the requirements of Section 4.6.1.2 of the Arkansas Nuclear One, Unit 2, Technical Specifications and 10CFR50, Appendix J(V)(B) and the submittal guidance of Regulatory Guide 10.1, by AP&L letter 2CAN088508 dated August 19, 1985.

During NRC Region IV inspection 313-368/87-32 conducted September 14-16, 1987, AP&L committed to revise the May 1985 Integrated Leak Rate Test (ILRT) Report to include a summary of the results using the Total Time Analysis method described in ANSI 45.4 - 1972, "Leakage Rate Testing of Containment Structures for Nuclear Reactors," and Bechtel Topical Report BN-TOP-1, "Testing Criteria for Integrated Leakage Rate Testing of Primary Containment Structures for Nuclear Power Plants." Although Total Time Analysis data were included in Appendix D - Trend Report, the results in the original report were summarized using the Mass-Point Analysis described in ANSI/ANS 56.8, 1981, "Containment System Leakage Testing Requirements." A specific Total Time Analysis has now been included in Appendix D. The results of the Total Time Analysis indicate a successful test as did the Mass-Point Analysis method.

Additional changes to the report have been made to address findings identified in NRC inspection report 50-368/8515. This inspection covered the containment integrated leakage rate test performed May 1985. The findings related to the Type C "as-found" tests results. Revisions to the ILRT report reflect the results of AP&L's review of and actions taken to resolve these findings.

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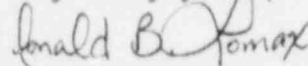
NRC unresolved item 368/8515-01 related to containment sump penetrations 2P-66 and 2P-67. These were also the subject of Licensee Event Report 368/88-001. In the original ILRT report, the results of "as-left" local leak rate (Type C) testing for these penetrations were included in the Type A leakage adjustment. In January 1988, while performing a review of containment piping penetrations with the assistance of industry experts in the area of design and testing, it was concluded that these penetrations do not require Type C testing. The changes to the ILRT report reflect the results of this review. These penetrations will continue to be leak tested during Type A testing (ILRT).

NRC unresolved item 368/8515-02 related to the Type C testing results for check valve 2CCW-38. The ILRT report has been revised to conform with the Technical Specifications, Table 3.6.1, "Containment Isolation Valves," for the penetration associated with this valve and other check valves not subject to Type C testing. The affected penetrations are 2P-39, 2P-40, 2P-51, and 2P-52. The measured leakage for the check valves has been deleted from the report. The test results were not affected by this change.

During the report revision process, a minor error was identified. A correction has been made to the "as-left" leakage for penetration 2P-14 to indicate the maximum pathway leakage rather than the minimum pathway leakage. This correction did not affect the results of the test.

Please incorporate the revised pages into the report as indicated by the page numbers.

Very truly yours,



for Dan R. Howard
Manager, Licensing

DRH:PLM:dm
enclosure

cc w/encl: U. S. Nuclear Regulatory Commission
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