James A. FitzPatrick Nuclear Power Plant 268 Lake Road P.O. Box 41 Lycoming, New York 13093

315-342-3840



Michael J. Colomb Site Executive Officer

IErra

October 27, 1998 JAFP-98-0349

United States Nuclear Regulatory Commission Attn: Document Control Desk Mail Station P1-137 Washington, D.C. 20555

Subject: Docket No. 50-333 LICENSEE EVENT REPORT: LER-98-011

> Failure to Meet Drywell Continuous Atmosphere Monitoring System Technical Specifications Surveillance Test Requirements

Dear Sir:

This report is submitted in accordance with 10 CFR 50.73(a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications."

There are no commitments contained in this report.

Questions concerning this report may be addressed to Mr. Gordon Brownell at (315) 349-6360.

Very truly yours,

MICHAEL J. COLOMB

MJC:GB:las Enclosure

cc: USNRC, Region 1 USNRC, Project Directorate USNRC Resident Inspector INPO Records Center

> 9811030101 981027 PDR ADDCK 05000333 S PDR

NRC FC (6-1998			NSEE EV			LER)	сомм	ISSION	Est req anx Ma	timated burden pa uest: 50 hrs. Re d fed back to indu nagement Branch	OMB NO. 315 ar response to comp ported lessons learning stry. Forward comme (T-6 F33), U.S. Nuc o the Paperwork	oly with this ed are incorport ants regarding clear Regulato	mandatory ii prated into ti burden estii ry Commissi	he licensing process mate to the Records on. Washington, DC	
			ee reverse f ligits/charad						Ma dis	nagement and Bu play a currently v	dget, Washington, Od alid OMB control nu required to respond to	C 20503. If a mber, the NR	in information C may not	conduct or sponsor.	
FACILITY	NAM	E (1)					An oracle second states		DO	OCKET NUMBE	R (2)	1	P	AGE (3)	
James	s A. F	itzPatric	Nuclear F	ower Pla	nt					0500033	33		1	OF 5	
Failure Requir	e To reme	Meet Dry	well Conti	nuous Atr	nospher	e Monito	oring S	System	Te	chnical Sp	pecifications	Surveil	lance 1	est	
EVE	NT DA	TE (5)	LEI	R NUMBER (6)	REP	ORTD	ATE (7)		1	OTHER FACI	LITIES IN	OLVED	(8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEA	R	FACILITY NA	ME		DOCKET NUMBER		
09	29	98	98	011	00	10	27	98		FACILITY NAME DOCKET NUMBER N/A 05000					
OPERA	TING	NI	THIS	REPORT IS	SUBMITT	ED PURSU	ANTT	O THE R	EQU	JIREMENTS	OF 10 CFR 5:	(Check or	ne or mor	e) (11)	
MODE	(9)	N	20.220			20.2203	Contraction of the Property of	Construction of the second sec		X 50.73	one entropy and an entropy and an entropy and and		Y	3(a)(2)(viii)	
POWER 100		100	20.2203(a)(1)			20.2203	3(a)(3)(i	i)		50.73		50.73(a)(2)(x)			
LEVEL	(10)	100	20.220)3(a)(2)(i)		20.2203	3(a)(3)(i	ii)		50.73	(a)(2)(iii)		73.71	1	
			20.220)3(a)(2)(ii)		20.2203	3(a)(4)		_	50.73	(a)(2)(iv)		OTHE	R	
			20.220	50.36(c)(1)				50.73(a)(2)(v)			Specify in Abstract below				
		20.2203(a)(2)(iv) 50.36(c				(c)(2)			50.73	(a)(2)(vi:		or in NRC Form 366A			
Non-ond surgranus					LICEN	SEE CONT	ACT F	OR THIS	LER	R (12)		Contraction and Advances and Contraction		Contract of the Source of Article Contracts	
NAME					Contraction of the A is also hid, includes					TELEPHONE N	UMBER I nolu te Ar	rea Code)			
Mr. G	ordor	Browne	II, Licensin	g Enginee	n						(315)	349-63	360		
				1	T		1	1		T	T	1			
CAUS	ε	SYSTEM	COMPONENT	MANUFAC	CTURER	REPORTABL TO EPIX	8	CAU	SE	SYSTEM	COMPONENT	MANUFA	CTURER	REPORTABLE TO EPIX	
						an, santan tigi, karanyan amanana	-								
		S	UPPLEMENT	AL REPORT	EXPECTER	D (14)		1		EXI	PECTED	MONTH	DAY	YEAR	
YE (If		complete EX	PECTED SUE	MISSION D	ATE).		XN	0		SUB	MISSION TE (15)				
ABSTR	ACT (Limit to 140	00 spaces, i.e	e., approxim	ately 15 si	ingle-space	d typev	written li	nes	(16)			COMPANY AND ADDRESS	and the second	
Prog requ Cont Spec freq chan	ram ire inu ifi uen nel TSs	, audi ments ous At cally, cy cal or in . At	tors ic for the mospher the so ibratic clude of the tim	dentifi e react cope an on test calibra ne of t	ed th for contorin d acc proc ation the di	at Teo olant eptance edure of the scove:	chni lea M) S ce c did a fu ry,	cal kage yste rite not ll i the	spd mi ri ns re	ecifica etectic were no a of th ncompas trument actor m	on Prote ations (on syste ot being ne once as the e range node swi ately 10	TS) s ms Dr met. per t ntire as de tch w	urvei ywell hree inst finec as in	months rument in the	

The cause for the deficient test procedure was poor managerial oversight of work performed by a contractor.

Scheduled corrective actions include: declaring both channels of the CAM System inoperable, obtaining periodic grab samples to monitor drywell activity, revising CAM System surveillance test procedures, retesting of the CAM System in accordance with TS requirements, conducting a root cause evaluation, and reviewing the results of the evaluation and lessons learned with Radiological and Environmental Services Department personnel.

power.

NRC FORM 366A (6-1998)		U.S.	NUCLEAR REC	BULATORY	COM	MISS	ION		
	E EVENT REPORT (LE	R)							
FACILITY NAME (1)	DOCKET (2) LER NUMBER (6)					PAGE (3)			
James A. FitzPatrick Nuclear Power Plant	05000333	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2	OF	5		
Sames A. Hizratiick Nuclear Fower Flant	05000333	98	011	00	-	UF	0		

TEXT (If more space is required, use additional copies of NRC Form 366A) [17]

EIIS Codes in []

1 .

EVENT DESCRIPTION

On September 29,1998, during the conduct of a scheduled Quality Assurance Department audit of FitzPatrick's Radiation Protection (RP) Program, it was identified that Technical Specifications (TS) surveillance requirements for the reactor coolant leakage detection systems [IJ] Drywell Continuous Atmosphere Monitoring (CAM) System [IL] were not being met. Specifically, the test scope and acceptance criteria contained in the procedure used to control TS required once per three months calibration frequency for the CAM System did not encompass the entire instrument channel or include calibration of the full instrument range as defined in TSs. At the time of the discovery, the reactor mode switch was in the RUN position and the plant was operating at approximately 100 percent rated power.

The Drywell CAM System utilizes a two-channel monitor to provide information on particulate and noble gas activities in the Drywell atmosphere. Two independent and redundant process lines detect airborne radioactivity. Each process line includes particulate, iodine, and gaseous detectors that have a controller, local audiovisual alarms, readout, and a commonly shared multipoint recorder. TS Table 4.6-2, "Minimum Test and Calibration Frequency for Continuous Atmosphere Monitoring System" requires that the air particulate and the gaseous activity analyzers be calibrated once per three month period.

Technical Specifications define channel calibration as the adjustment of channel output such that it responds within the necessary range and accuracy to known values of the parameter that the channel monitors. The channel calibration shall encompass the entire channel, including the required sensor, alarm, display, and trip functions. The channel calibration may be performed by means of any series of sequential, overlapping, or total channel steps so that the entire channel is calibrated.

The Radiological and Environmental Services (RES) Department uses procedure RP-RESP-03.01, "Drywell Constant Air Monitor" to fulfill TS Table 4.6-2 calibration requirements. However, test methods and parameters contained in RP-RESP-03.01 do not meet instrument calibration requirements as defined. Once per three months, the particulate and gaseous detectors are "efficiency" calibrated using standard calibrated radiation sources to attain response efficiencies. This test measures only a partial range of the instrument and does not include alarm response verification. NRC FORM 366A (6-1998)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

		TEXT	CONT	INUAT	ION
--	--	------	------	-------	-----

FACILITY NAME (1)	DOCKET (2)	LER NUMBER (6)				PAGE (3)		
James A. FitzPatrick Nuclear Power Plant	05000333	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2	05	5	
	00000333	98	011	00	3	UP	0	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT DESCRIPTION (cont.)

An electronic calibration, as defined by the TSs definition of an instrument channel calibration, however, is performed by the Instrument and Control(I&C) Department on two year frequency intervals within the I&C Preventive Maintenance (PM) Program. This test uses electronic inducted signals that test the full range of the instrument channel and verifies alarm response. This electronic calibration was last performed in January 1998.

EVENT CAUSE

In 1994, a surveillance test (ST) adequacy review was completed, which included RES Department procedures. Identified during the review was the condition that CAM surveillance tests may not have been in compliance with TS requirements. A procedure change request (PCR) was generated to recommend procedure revisions to establish TS compliance. The PCR was found unprocessed during the September 1998 RP Program audit.

The cause for the uncorrected calibration procedure was poor managerial oversight of work performed by a contractor. Specifically, there existed a lack of ownership by RES management over the work being completed by contracted personnel hired by the Authority to perform various RES Department technical support tasks. This resulted in:

- the failure to appropriately identify and report nonconforming conditions through the plant's Deviation and Event Report system, and
- inadequate communications as evidenced by the failure of RES supervision to be aware of potential TS concerns and the failure of the turnover process at the completion of the work assignment by the contractors prior to their exiting the plant. No historical data was found showing that the PCR was ever entered into the procedure change control process.

ANALYSIS

This report is being submitted in accordance with 10 CFR 50.73 (a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications."

NRC FORM 366A

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET (2)	LI	PAGE (3)			
James A. FitzPatrick Nuclear Power Plant	05000333	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	4 05	6
	05000333	98	011	00	4 UF	0

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

ANALYSIS (cont.)

TS Section 3.6, BASES, states that the reactor coolant leakage detection systems consist of the Drywell Sump Monitoring System and the Drywell CAM System. The CAM System supplements the Drywell Sump Monitoring System in detecting abnormal leakage that could occur from the Reactor Coolant System. The Safety Related function of detecting Drywell leakage is performed redundantly by the primary containment sump system and its alarm systems, and the Drywell temperature indication and alarm system. The Drywell CAM System is non-safety related and only acts as a backup to these systems.

Both channels of the CAM System were declared inoperable on October 01, 1998 following confirmation that TS calibration requirements were not being met. Operators immediately entered TS Section 3.6.D.6 Limiting Condition For Operation (LCO) which states that with the CAM System (gaseous or particulate analyzers) inoperable, plant operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours. As a result, the safety significance of this event was minimal.

CORRECTIVE ACTIONS

- A root cause evaluation was completed for this event. This evaluation and lessons learned will be reviewed with RES Department personnel. (Scheduled Completion Date - December 15, 1998)
- 2. Subsequent to this 1994 event, FitzPatrick had issued Administrative Procedure AP-01.08, "Control Of Contractor and Vendor Activities". This procedure provides the requirements for the controls of contractor and vendor activities and guidelines for management expectations regarding contractor and vendor oversight.
- Calibration activities for both channels of the CAM System's gaseous and particulate radiation monitors were successfully completed on October 09, 1998.
- 4. RES procedure RP-RESP-03.01 will be revised prior to its next scheduled test due date to: (1) include the full instrument range electronic calibration; and (2) verify radiation monitor alarm response. (Scheduled Completion Date - December 15, 1998)

NRC FORM 366A

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

, FACILITY NAME (1)	DOCKET (2)	LER NUMBER (6)				PAGE (3)		
James A. FitzPatrick Nuclear Power Plant	05000333	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	E	05		
	05000333	98	011	00	D	Ur	9	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

CORRECTIVE ACTIONS (cont.)

5. RES Department will evaluate the results of other ST adequacy reviews conducted during the 1994 period to ensure inconsistencies and/or deficiencies previously identified have been resolved. (Scheduled Completion Date - March 31, 1999)

ADDITIONAL INFORMATION

A. Previous similar Events:

LERs 97-009, 97-006, 95-014, 95-012 and 94-003 dealt with exceeding or not performing surveillance test at required frequencies. However, the causes for those previous occurrences were not similar, therefore, the corrective actions would not have precluded this event.

B. Extent of Condition:

An extent of condition review will be conducted by the RES Department of previous contractor activities associated with ST adequacy reviews as discussed in Corrective Action number 5. Additionally, other departments will also evaluate results of a select number previously conducted ST adequacy reviews to provide assurance that similar conditions do not exist.