

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MARCH, 1986

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

March 1 - 31

March 1, 0001 Hours. The Unit entered March with the reactor subcritical and unit off-line in cold shutdown for first refuel outage.

March 31, 2400 Hours. Reactor in cold shutdown for first refueling outage.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED
MAINTENANCE.

1. Amendments to facility license or Technical Specification.

AMENDMENT 35 - This revised the Unit #1 specification for High
Radiation Door Control.

AMENDMENT 36 - This revised the Unit #1 specification for Main
Steam Line Low Pressure Response Time.

2. Facility or procedure changes requiring NRC approval.

There were no Facility or Procedure Changes Requiring NRC
approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during
this reporting period.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of
safety-related maintenance completed on Unit One during the
reporting period. The headings indicated in this summary
include: Work Request number, Component Name, Cause of
Malfunction, Results and Effects on Safe Operation, and
Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L35151	S/D Cooling Outboard Valve 1E12-F008	Valve leaks by seat.	S/D cooling suction header HI pressure alarm present.	Repaired valve internals.
L38796	LPCS Discharge Alarm Switches	M-1-1-84-088.	Split up high and low pressure alarms on LPCS pump discharge pressure.	Completed under ECN-ED-147 and PFL-172 within M-1-1-84-088.
L48950	B SBLC Pump Discharge Check Valve 1C41-F033B	Check valve would not seat properly.	Failure of LOS-SC-M1, check valve leaks.	Disassembled, cleaned and reassembled check valve, also seal welded bonnet.
L50629	B RHR Testable Check Valve 1E12-F041B	Bad air cylinder.	Valve would not cycle from control switch.	Replaced air cylinder and relubed.
L51401	A RHR Full Flow Test Stop Valve 1E12-F024A	Considerable leakage through valve packing.	Failure of LTS-300-7.	Repacked valve.
L51995	D Main Steam Line Rad Monitor 1D18-K610D	Component drawer was out of tolerance low.	D Rad monitor is reading low.	Repaired internal circuitry.
L52565	VE Damper 0VE-03YA	Damper actuator discon- nected and wired open.	During LIS-VC-053, damper would not stroke.	Reconnected linkage and unwired the damper.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L53920	A D/W Spray Upstream isolation stop valve 1E12-F016A	Valve had excessive leak- age through seat. Seat full of scratches.	Failure of LTS-100-43.	Machined valves disc and seats and reassembled.
L55044	C RHR WS Pump 1E12-C300C	Motor burned up.	WS pump inoperable.	Rewound motor and rein- stalled.
L55546	Mechanical snubbers HP20-15075, HP20-14025 and MS14-10485	Snubbers found broken.	Failed during functional testing.	Replace with new snubbers.
L55573	B RHR Steam Line Isolation Valve 1E12-F052B	Bad motor.	During LES-EQ-112 found motor grounded.	Replaced motor.
L55596	A RHR Valve 1E12-F068A	Failed motor during testing.	Valve inoperable.	Installed new motor.
L55709	B RHR Full Flow Test Stop Valve 1E12-F024B	Considerable leakage through valve packing.	Packing leaks when valve strokes.	Repacked valve.
L55920	B VC Refrigeration Unit 0VC05CB	Bad flair coupling on hot gas bypass valve.	Trips on low suction pressure.	Repaired flaired coupl- ings on hot gas bypass valves.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L56101	HPCS Min Flow Valve 1E22-F012	Bad motor.	Valve won't cycle from control switch.	Replaced motor.
L56142	B RHR HX Vent Valve 1E12-F074B	Torque switch close setting to low.	Valve will not completely close.	Changed torque switch close setting from 1 to 2-1/2.
L56231	Mechanical Snubber HP08-10245	Failed snubber.	Failed LTS-500-14, functional test.	Replaced with new snubber.
L56353	B RHR Testable Check Valve 1E12-F041B	Disc is not traveling into seat due to packing gland binding.	Valve will not seat to allow pressurization for LLRT.	Repacked and reassembled.
L56467	Control Room HVAC Detector OXY-VC-125B	Bad flowswitch and broken wires.	No indication on flow lamp with appropriate flow present.	Replaced flowswitch and repaired broken wires.
L56583	H ₂ Recombiner Valve 1HG001B	Bad packing.	Valve is binding when manually stroked and high amps when electrically stroked.	Repacked valve.
L56670	D/W Humidity Monitored Suction Valve 1CM017A	Bad terminal block and damaged field wires.	No indication when valve is full closed.	Replaced cable, lugs and terminal block.
L56890	H ₂ Recombiner Valve 1HG001A	Bad packing.	Valve hard to turn manually and draws high amperage when electrically stroked.	Repacked valve.
L56913	HPCS Standby Water Leg Pump 1E22-C003	Bad breaker.	Pump breakers trip on magnetics.	Replaced breaker.
L57132	A RHR Injection Valve 1E12-F042A.	Burned overload relay.	Valve has no light indication and cannot be cycled from control room.	Replaced burned overload relay.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, March 1 through March 31, 1986. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
86-005-00	02/17/86	ESF actuation from VR due to EO not completing procedure step by step.
86-006-00	02/18/86	Reactor water conductivity monitor inoperable due to no sample taken.
86-007-00	02/26/86	Missing O-ring on VQ valve due to improper installation during construction.
86-008-00	02/26/86	HPCS DG start from instrument pressure surge due to air in SBLC injection line.
86-009-00	02/25/86	VR outboard Group IV isolation due to alligator clip falling off terminal during RPS bus transfer.
86-010-00	03/04/86	Group IV PCIS isolation due to short to ground from jumper falling off terminal.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE 3/10/86
 COMPLETED BY James P. Peters
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: March 1986 GROSS HOURS IN REPORTING PERIOD: 744
 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt):3323 MAX DEPEND CAPACITY (MWe-Net):1036 DESIGN ELECTRICAL RATING (MWe-Net):1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY): N/A
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|--|---------------|---------------|-----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>0.0</u> | <u>0.0</u> | <u>12039.00</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>1642.00</u> |
| 7. HOURS GENERATOR ON LINE | <u>0.0</u> | <u>0.0</u> | <u>11642.00</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>0.0</u> | <u>0.0</u> | <u>32213650</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>0.0</u> | <u>0.0</u> | <u>10499394</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>-11041</u> | <u>-25470</u> | <u>9978987</u> |
| 12. REACTOR SERVICE FACTOR | <u>0.0%</u> | <u>0.0%</u> | <u>61.0%</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>0.0%</u> | <u>0.0%</u> | <u>69.3%</u> |
| 14. UNIT SERVICE FACTOR | <u>0.0%</u> | <u>0.0%</u> | <u>59.0%</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>0.0%</u> | <u>0.0%</u> | <u>59.0%</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>-1.4%</u> | <u>-1.1%</u> | <u>48.8%</u> |
| 17. UNIT CAPACITY FACTOR(USING DESIGN MWe) | <u>-1.4%</u> | <u>-1.1%</u> | <u>46.9%</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>0.0%</u> | <u>0.0%</u> | <u>17.4%</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
 The First Refueling, Maintenance, Surveillance and Modification Outage began October 18, 1985 and will last into May, 1986.
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: May 3, 1986

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: 3/10/86
COMPLETED BY: James P. Peters
TELEPHONE: (815) 357-6761
MONTH: MARCH 1986

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____	-15	17. _____	-15
2. _____	-14	18. _____	-16
3. _____	-13	19. _____	-16
4. _____	-15	20. _____	-16
5. _____	-16	21. _____	-16
6. _____	-13	22. _____	-16
7. _____	-16	23. _____	-16
8. _____	-16	24. _____	-16
9. _____	-14	25. _____	-15
10. _____	-14	26. _____	-15
11. _____	-14	27. _____	-15
12. _____	-14	28. _____	-15
13. _____	-15	29. _____	-14
14. _____	-16	30. _____	-14
15. _____	-15	31. _____	-14
16. _____	-15		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MARCH 1986

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE 3/10/86
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

NO.	DATE	TYPE	DURATION (HOURS)	REASON	METHOD OF	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED			SHUTTING DOWN THE REACTOR OR REDUCING POWER	

There were no Unit shutdowns or power reductions during this reporting period due to the refuel outage in progress.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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There were no Safety Relief Valves Operated for Unit One during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
0-64-86	O DG	Lube coupling and clean injection screen
0-65-86	O DG	Fix cylinder temperature sensor
0-67-86	O DG Cooling Water	Lube Coupling
0-68-86	O DG	Replace K55 Relay
1-424-86	1B DG	Clean Ejector Screen
1-425-86	1B DG	Relay Calibration
1-426-86	B RHR	HX Hydro
1-429-86	1E22-F010	LES-EQ-112
1-431-86	1E12-F068A	Test Thermal Overloads
1-432-86	1E12-F090A	Keep Closed
1-433-86	1E12-F092A	Keep Closed
1-437-86	1E12-F063C	Fill Suppression Pool
1-440-86	1E12-F009	Remove Packing
1-441-86	B/C RHR Seal Coolers	Isolate For Hydro
1-442-86	LPCS Waterleg Pump	Prevent Filling A RHR
1-445-86	1E12-F041B	Repair
1-450-86	1E51-F013	LES-EQ-112
1-456-86	1E12-F372C	Clean Sight Glass
1-460-86	1E12-F063C	To Fill Reactor Well
1-471-86	1E51-F031	LES-EQ-112
1-472-86	1E12-F026A	Install Limiter Plate
1-477-86	B RHR PRM	Isolate till MOD Complete
1-486-86	1E12-F073A	Valve Position Indication
1-489-86	1E22-C003	Lubricate
1-490-86	1E22-F010/23	LES-EQ-112
1-493-86	1E12-F063C	Add Water to Supp. Pool
1-495-86	HPCS	Suction Relief Hydro Bound.
1-497-86	LPCS/CY Spool Piece	Remove Spool Piece
1-500-86	1B DG	Replace Upper Head Gasket
1-503-86	1E12-F041A/B/C	Replace Cable
1-504-86	1E12-F092A	Replace Cable
1-527-86	1E51-N010	Replace Level Switch
1-528-86	1E51-N010	Support Level Switch Replacement
1-530-86	HPCS Waterleg Pump	Troubleshoot
1-539-86	1E22-F023	LES-EQ-112
1-543-86	RCIC Interlocks	Revise Wiring
1-556-86	1E21-C001	Inspect Coupling
1-565-86	A RHR WS	Relocate PRM Probe
1-569-86	1E12-F016A	Packing Leak
1-574-86	1E21-E003	Repair Wire in Breaker
1-579-86	1E12-F041C	Repair Sealtite at Limit Switch

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-580-86	1E12-F008	Hydro
1-581-86	1E12-F099B	Install Redundant Fault Protection
1-589-86	RHR S/D Cool Flow Switch	LST-86-47
1-602-86	1E12-F040A	Repair Ground Wires
1-603-86	1E12-F073A	Repair Conduit
1-604-86	1E12-F074A	Repair Conduit
1-606-86	1E12-F042B	Repair Contactor
1-608-86	1E51-F080	Repack Valve
1-622-86	1E12-F042A	Replace Thermal O/L
1-623-86	1E12-F332B	Admin. Control
1-625-86	1E51-F086	Repack Valve
1-633-86	1E12-C300A	Install Motor

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

MARCH 1986

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NP7-18

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 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

March 1-31

- March 1, 0001 Hours. The Unit entered March with the Reactor critical at 98% power and Generator on line at 1097 MWe. Also, power is being reduced to 89% (985 MWe) for surveillance testing.
- March 1, 0815 Hours. Reactor power reduced to 78% (850 MWe) due to EGC testing.
- March 2, 1000 Hours. Reactor Power increased to 100% (1122 MWe).
- March 2, 2100 Hours. Commenced Reactor shutdown to repair reactor water clean-up isolation valve IG33-F001.
- March 3, 0543 Hours. Reactor scram on IRM HI-HI.
- March 5, 1340 Hours. Reactor critical.
- March 5, 2135 Hours. Generator synchronized to the grid, began increasing power.
- March 8, 0400 Hours. Reactor power increased to 78% (870 MWe).
- March 8, 0600 Hours. Reactor power reduced to 51% (570 MWe) due to control rod manipulations.
- March 8, 1212 Hours. Reactor power reduced to 38% (456 MWe) due to spurious RR pumps down shift.
- March 8, 1305 Hours. Reactor power increased to 57% (625 MWe) due to upshifting RR pumps and continued power ascension.
- March 9, 2200 Hours. Reactor power decreased to 54% (605 MWe) for control rod manipulations.
- March 10, 0030 Hours. Final rod pattern set and began power ascension.
- March 12, 1800 Hours. Reactor power increased to 98% (1081 MWe) limited by flow control line.

II. MONTHLY REPORT FOR UNIT TWO (Continued)

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

March 1-31

- March 16, 0720 Hours. Reactor power reduced to 74% (835 MWe) for control rod manipulations.
- March 18, 0700 Hours. Reactor power increased to 100% (1108 MWe) full power.
- March 18, 2050 Hours. Reactor power reduced to 95% (1065 MWe) for preconditioning.
- March 19, 1000 Hours. Reactor power at 100% (1110 MWe) limited by Xenon.
- March 20, 2400 Hours. Reactor power reduced to 94% (1040 MWe) for control rod manipulations.
- March 21, 1500 Hours. Reactor power at 100% (1116 MWe) full power.
- March 27, 1900 Hours. Reactor power reduced to 71% (800 MWe) for control rod manipulation.
- March 29, 2300 Hours. Reactor power at 100% (1110 MWe) full power.
- March 31, 2400 Hours. Reactor power at 99% (1097 MWe) full power.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

AMENDMENT 19 - This revised Unit #2 specification for High Radiation Door control.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure change requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L56163	APRM-C LPRM 24-57B	Bad ICPS	ICPS power is at 20.4 VDC and should be at 100 VDC.	Replaced ICPS and set at 100 VDC.
L56188	Div-I Post Loca Chart Recorder 2B21-R884A	Bad chart drive circuit board and motor.	Chart drive does not work.	Replaced chart drive circuit board and motor.
L56270	TSV closed scram Relay 2C71A-K10C	EQ penetration lead wire grounded at limit switch.	Continuously blows fuses.	Put heat shrink on entire length of wire.
L56400	RWCU inboard isolation valve 2G33-F001	Bound up declutch lever	Valve failed to open after isolation and breaker was re-energized.	Declutched valve operator and opened valve.
L56409	LPCS min flow valve 2E21-F011	Flow switch 2E21-N004 out of calibration.	Sporadic valve positioning.	Recalibrated flow switch 2E21-N004.
L56461	RWCU outlet isolation valve 2G33-F004	Dirty contacts and loose connections on torque switch.	Valve does not open electrically.	Cleaned contacts and tightened screws on torque switch.
L56537	SBGT damper actuator 2FZ-VG003.	Bad mA source to actuator	Actuator 2FZ-VG003 for damper 2VG002 is cycling open and closed.	Installed new mA source.
L56570	Control Room Panel 2PM18J ground 211-CB21.	Bad Power Supply.	120 Volt ground on 211Y CB21	Replaced with new power supply.
L56897	SBGT moist. sep. diff. press. alarm 2PDS-VG021.	Pressure switch out of tolerance low	Alarm is up when train is in operation.	Recalibrated diff. press. switch.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, March 1 through March 31, 1986. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
86-002-00	02/06/86	RCIC System and 1E51-F008 inoperable due to loose contact strip in contact block.
86-003-00	02/17/86	Inadvertently placed mode switch into run due to inattentiveness.
86-004-00	02/16/86	Loss of main transformer due to phase to phase fault on main transformers.
86-005-00	03/03/86	Scram on IRM HI-HI alarm due to instrumentation sensing feedwater regulator valve shut when 15% open.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LaSalle Two
 DATE 3/10/86
 COMPLETED BY James P. Peters
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: March, 1986 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY): N/A

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>688.05</u>	<u>2056.59</u>	<u>7445.99</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>29.83</u>	<u>29.83</u>
7. HOURS GENERATOR ON LINE	<u>680.14</u>	<u>2029.44</u>	<u>7265.74</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>1989000</u>	<u>5953104</u>	<u>21461656</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>665452</u>	<u>1991323</u>	<u>7101392</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>643693</u>	<u>1933582</u>	<u>6756797</u>
12. REACTOR SERVICE FACTOR	<u>92.5%</u>	<u>95.2%</u>	<u>58.5%</u>
13. REACTOR AVAILABILITY FACTOR	<u>92.5%</u>	<u>96.6%</u>	<u>58.8%</u>
14. UNIT SERVICE FACTOR	<u>91.4%</u>	<u>93.9%</u>	<u>57.1%</u>
15. UNIT AVAILABILITY FACTOR	<u>91.4%</u>	<u>93.9%</u>	<u>57.1%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>83.5%</u>	<u>86.4%</u>	<u>51.2%</u>
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>80.3%</u>	<u>83.0%</u>	<u>49.3%</u>
18. UNIT FORCED OUTAGE RATE	<u>8.6%</u>	<u>6.0%</u>	<u>22.6%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): There are no shutdowns scheduled over the next 6 months.			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP <u>N/A</u>			

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: 3/10/86
COMPLETED BY: James P. Peters
TELEPHONE: (815) 357-6761
MONTH: March 1986

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____	928	17. _____	983
2. _____	1021	18. _____	1063
3. _____	29	19. _____	1060
4. _____	-14	20. _____	1062
5. _____	-6	21. _____	1051
6. _____	480	22. _____	1063
7. _____	744	23. _____	1058
8. _____	661	24. _____	1053
9. _____	760	25. _____	1049
10. _____	761	26. _____	1045
11. _____	943	27. _____	996
12. _____	1025	28. _____	896
13. _____	1026	29. _____	1029
14. _____	1020	30. _____	1064
15. _____	1015	31. _____	1058
16. _____	897		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE 3/10/86
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

REPORT MONTH MARCH 1986

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
17	3/3/86	F		64.27	A-Feedwater Reg. valve instrumentation out of calibration.	3	Reactor scram on IRM HI-HI

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES</u> <u>ACTUATED</u>	<u>NO & TYPE</u> <u>ACTUATIONS</u>	<u>PLANT</u> <u>CONDITION</u>	<u>DESCRIPTION</u> <u>OF EVENT</u>
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There were no safety relief valves operated for Unit #2 during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-89-86	HPCS Petter DG Air Compressor.	Replace high pressure valves.
2-93-86	2E21-F011.	Inoperable per Tech. Spec.
2-103-86	2B DG Air Compressor.	Replace belts.
2-109-86	RHR S/D Cooling System.	LIS-RH-412.
2-124-86	2B HPCS DG.	Change filter and lube.
2-125-86	2B HPCS DG.	Change location of sensor.
2-126-86	2B HPCS DG.	Lubing and filter changes.
2-138-86	2E51-C003.	Adjust impeller.
2-139-86	A RHR WS strainer backwash control.	Change wiring.
2-148-86	RCIC.	Inspect water leg pump min. flow orifice.
2-149-86	RCIC.	Calibrate RCIC pump disch. flow instrument.
2-150-86	RCIC.	Calibrate drain pot level switch.
2-156-86	2E12-F023/99A/99B/53A 53B/8/9.	LIS-NB-411.

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the radioactive waste treatment system during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

April 8, 1986

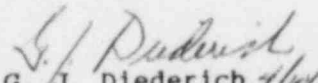
Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period March 1, 1986 through March 31, 1986.

Very truly yours,


G. J. Diederich
Station Manager
LaSalle County Station

GJD/RJR/bke

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
L. J. Anastasia, PIP Coordinator SNED
J. E. Ellis, GE Resident
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