



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY, ET AL

DOCKET NO. 50-461

CLINTON POWER STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 11  
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that
  - A. The application for amendment by Illinois Power Company\* (IP), Soyland Power Cooperative, Inc., and Western Illinois Power Cooperative, Inc. (the licensees) dated May 18, 1988, as supplemented on June 2, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

\*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and Western Illinois Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

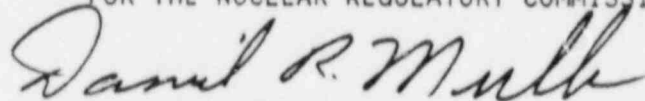
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 11, are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director  
Project Directorate III-2  
Division of Reactor Projects - III,  
IV, V and Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 29, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 11

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
2-3	2-3
2-4	2-4
3/4 3-21	3/4 3-21
3/4 3-24	3/4 3-24

TABLE 2.2.1-1  
REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

CLINTON - UNIT 1	<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
	1. Intermediate Range Monitor		
	a. Neutron Flux-High	< 120/125 divisions of full scale	< 122/125 divisions of full scale
	b. Inoperative	NA	NA
	2. Average Power Range Monitor:		
	a. Neutron Flux-High, Setdown	< 15% of RATED THERMAL POWER	< 20% of RATED THERMAL POWER
	b. Flow Biased Simulated Thermal Power-High		
	1) Flow Biased	< 0.66 (W-ΔW)+48%, (a) with a maximum of < 111.0% of RATED THERMAL POWER	< 0.66 (W-ΔW)+51%, (a) with a maximum of < 113.0% of RATED THERMAL POWER
	2) High Flow Clamped		
	c. Neutron Flux-High	< 118% of RATED THERMAL POWER	< 120% of RATED THERMAL POWER
	d. Inoperative	NA	NA
	3. Reactor Vessel Steam Dome Pressure - High	≤ 1065 psig	≤ 1080 psig
	4. Reactor Vessel Water Level - Low, Level 3	> 8.9 inches above Instrument zero*	> 8.3 inches above instrument zero
	5. Reactor Vessel Water Level-High, Level 8	≤ 52.0 inches above Instrument zero*	< 52.6 inches above Instrument zero
	6. Main Steam Line Isolation Valve - Closure	≤ 8% closed	≤ 12% closed
	7. Main Steam Line Radiation - High	< 3.0 x full power background**	< 3.6 x full power background**
	8. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig

2-2

Amendment No. 11

TABLE 2.2.1-1 (Continued)  
 REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
9. Scram Discharge Volume Water Level - High		
a. Level Transmitter		
1C11-N601A	≤ 30 in.†	≤ 40 1/4 in.
1C11-N601B	≤ 30 in.†	≤ 40 1/4 in.
1C11-N601C	≤ 30 in.††	≤ 39 3/16 in.
1C11-N601D	≤ 30 in.††	≤ 39 3/16 in.
b. Float Switch		
1C11-N013A	≤ 762 ft. 1.375 in. msl	≤ 763 ft. 3 1/4 in. msl
1C11-N013B	≤ 762 ft. 1.125 in. msl	≤ 763 ft. 3 1/4 in. msl
1C11-N013C	≤ 762 ft. 0.75 in. msl	≤ 763 ft. 1 11/16 in. msl
1C11-N013D	≤ 762 ft. 1.125 in. msl	≤ 763 ft. 1 11/16 in. msl
10. Turbine Stop Valve - Closure	≤ 5% closed	≤ 7% closed
11. Turbine Control Valve Fast Closure, Valve Trip System Oil Pressure - Low	> 530 psig NA	> 465 psig NA
12. Reactor Mode Switch Shutdown Position	NA	NA
13. Manual Scram	NA	NA

(a) The Average Power Range Monitor Scram Function varies as a function of recirculation loop drive flow (W).  $\Delta W$  is the difference in indicated drive flow (in percent of drive flow which produces the same core flow) between two loop and single loop operation at the same core flow.  $\Delta W = 0$  for two loop operation.  $\Delta W = 8\%$  for single loop operation.

\*See Bases Figure B 3/4 3-1.

†Instrument zero is 759 ft. 11 in. msl

††Instrument zero is 759 ft. 10.5 in. msl

\*\*Within 24 hours prior to the planned start of the hydrogen injection test, with reactor power at greater than 20% of RATED THERMAL POWER, the normal full power background radiation level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip setpoints may be adjusted during the test based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be verified and the associated trip setpoints shall be returned to their normal value within 24 hours of re-establishing normal radiation levels after completion of the hydrogen injection test at greater than 20% of RATED THERMAL POWER or within 12 hours of establishing reactor power levels below 20% of RATED THERMAL POWER.

TABLE 3.3.2-2 (Continued)  
CRVICS INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. <u>PRIMARY AND SECONDARY CONTAINMENT ISOLATION (Continued)</u>		
k. Containment Pressure - High	$\leq 2.62 \mu\text{sid}$	$\leq 3.00 \text{ psid}$
l. Main Steam Line Radiation - High	$\leq 3.0 \times \text{full power background}^\#$	$\leq 3.6 \times \text{full power background}^\#$
m. Fuel Building Exhaust Radiation - High	$\leq 10 \text{ mR/hr}$	$\leq 17 \text{ mR/hr}$
n. Manual Initiation	NA	NA
2. <u>MAIN STEAM LINE ISOLATION</u>		
a. Reactor Vessel Water Level - Low Low Low, Level 1	$\geq -145.5 \text{ in.}^*$	$\geq -147.7 \text{ in.}$
b. Main Steam Line Radiation - High	$\leq 3.0 \times \text{full power background}^\#$	$\leq 3.6 \times \text{full power background}^\#$
c. Main Steam Line Pressure - Low	$\geq 849 \text{ psig}$	$\geq 837 \text{ psig}$
d. Main Steam Line Flow - High	$\leq 170 \text{ psid}^{**}$	$\leq 178 \text{ psid}^{**}$
e. Condenser Vacuum - Low	$\geq 8.5 \text{ in. Hg vacuum}$	$\geq 7.6 \text{ in. Hg vacuum}$
f. Main Steam Line Tunnel Temp. - High	$\leq 165^\circ\text{F}$	$\leq 176^\circ\text{F}$
g. Main Steam Line Tunnel $\Delta$ Temp. - High	$\leq 54.5^\circ\text{F}$	$\leq 60^\circ\text{F}$
h. Main Steam Line Turbine Bldg. Temp. - High		
(1) 1E31 - N559 A, B, C, D	$\leq 131.2^\circ\text{F}$	$\leq 138^\circ\text{F}$
1E31 - N560 A, B, C, D		
1E31 - N561 A, B, C, D		
1E31 - N562 A, B, C, D		
(2) 1E31 - N563 A, B, C, D	$\leq 143.2^\circ\text{F}$	$\leq 150^\circ\text{F}$
i. Manual Initiation	NA	NA
3. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>		
a. $\Delta$ Flow - High	$\leq 59 \text{ gpm}$	$\leq 66.1 \text{ gpm}$
b. $\Delta$ Flow Timer	$\geq 45 \text{ sec.}$	$\leq 47 \text{ sec.}$

TABLE 3.3.2-2 (Continued)

CRVICS INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
5. <u>RHR SYSTEM ISOLATION (Continued)</u>		
c. Reactor Vessel Water Level - Low, Level 3	$\geq 8.9$ in.*	$\geq 8.3$ in.
d. Reactor Vessel Water Level - Low Low Low, Level 1	$\geq -145.5$ in.*	$\geq -147.7$ in.
e. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	$\leq 135$ psig**	$\leq 150$ psig**
f. Drywell Pressure - High		
1) Containment Spray	$\leq 1.68$ psig	$\leq 1.88$ psig
2) Fuel Pool Cooling	$\leq 1.68$ psig	$\leq 1.88$ psig
g. Manual Initiation	NA	NA

\*See Bases Figure B 3/4 3-1.

\*\*Initial setpoint. Final setpoint to be determined during startup test program. Any required change to this setpoint shall be submitted to the Commission within 90 days of test completion.

#Within 24 hours prior to the planned start of the hydrogen injection test, with reactor power at greater than 20% of RATED THERMAL POWER, the normal full power background radiation level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip setpoints may be adjusted during the test based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be verified and the associated trip setpoints shall be returned to their normal value within 24 hours of re-establishing normal radiation levels after completion of the hydrogen injection test at greater than 20% of RATED THERMAL POWER or within 12 hours of establishing reactor power levels below 20% of RATED THERMAL POWER.