CP&L

Carolina Power & Light Company
P.O. Box 10429
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OCT 27 1998

10 CFR 50.46(a)(3)(ii)

SERIAL: BSEP 98-0198

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 AND 50-324/LICENSE NOS. DPR-71 AND DPR-62
ANNUAL REPORT OF EMERGENCY CORE COOLING SYSTEM EVALUATION MODEL CHANGES AND ERRORS

Gentlemen:

In accordance with 10 CFR 50.46(a)(3)(ii), Carolina Power & Light (CP&L) Company hereby submits the annual report summarizing the effect of changes and errors in accepted loss-of-coolant accident (LOCA) Emergency Core Cooling System (ECCS) evaluation models applicable to the Brunswick Steam Electric Plant (BSEP), Unit Nos. 1 and 2.
10 CFR 50.46(a)(3)(i) requires each licensee to estimate the impact of changes and errors in accepted ECCS evaluation models or in the application of these models. 10 CFR 50.46(a)(3)(ii) specifies reporting requirements based on the sum of the absolute value of the changes and errors in calculated peak cladding temperature.

The enclosure of this letter provides the annual update of the analysis changes and errors in the LOCA ECCS evaluation model applicable to BSEP. A report of analysis changes and errors in the LOCA ECCS evaluation model for BSEP was previously submitted by CP&L's letter dated February 25, 1998 (Serial: BSEP 98-0045). There have been no additional changes or errors since the February 25, 1998, report. The absolute value of the analysis changes and errors in the LOCA ECCS evaluation model remains approximately 23° F for GE7 fuel and 10° F for GE13 fuel. These values represent the impact of peak cladding temperature changes and errors through June 30, 1998.

No regulatory commitments are being made in this letter. Please refer any questions regarding this submittal to Mr. Warren J. Dorman, Supervisor - Licensing, at (910) 457-2068.

Sincerely,

Keith R. Jury
Manager - Regulatory Affairs
Brunswick Steam Electric Plant

WRM/wrm

Enclosure: Update To Report Of Loss-of-Coolant Accident (LOCA) Emergency Core Cooling System (ECCS) Evaluation Model Analysis Changes and Errors

cc (with enclosure):

U. S. Nuclear Regulatory Commission, Region II
ATTN: Mr. Luis A. Reyes, Regional Administrator
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, GA 30303

U. S. Nuclear Regulatory Commission
ATTN: Mr. Charles A. Patterson, NRC Senior Resident Inspector
8470 River Road
Southport, NC 28461-8869

U. S. Nuclear Regulatory Commission
ATTN: Mr. David C. Trimble, Jr. (Mail Stop OWFN 14H22)
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Ms. Jo A. Sanford
Chair - North Carolina Utilities Commission
P.O. Box 29510
Raleigh, NC 27626-0510

ENCLOSURE

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 AND 50-324/LICENSE NOS. DPR-71 AND DPR-62
ANNUAL REPORT OF EMERGENCY CORE COOLING SYSTEM
EVALUATION MODEL CHANGES AND ERRORS

UPDATE TO REPORT OF LOSS-OF-COOLANT ACCIDENT (LOCA)
EMERGENCY CORE COOLING SYSTEM (ECCS)
EVALUATION MODEL ANALYSIS CHANGES AND ERRORS

SAFER/GESTR LOCA ECCS EVALUATION MODEL ANALYSIS CHANGE/ERROR SUMMARY

Change or Error Notice	Notice or Document Date	Period Covered	Change or Error Description	General Electric BWR Estimated Peak Cladding Tempearture (PCT) Impact	Estimated Brunswick Steam Electric Plant (BSEP) PCT Impact and Estimated PCT	Cumulative BSEP PCT Change	Is Cumulative BSEP PCT Change Greater Than 50° F
MFN 032-98	6/30/98 (Received 7/24/98)	6/28/97-6/30/98	No changes or errors.	N/A	N/A U1 1556° F GE7 U2 1560° F GE7 U1 1545° F GE13 U2 1545° F GE13	23° F GE7 10° F GE13	Incremental - No Cumulative - No

PCT CHANGE SUMMARY THROUGH JUNE 30, 1998

Unit and Fuel Type	Greatest PCT Reported in a LOCA Document Submitted to NRC.	Current Estimated PCT	Change From Reported PCT
BSEP1 GE7*	1533° F	1556° F	+23° F
BSEP1 GE13	1535° F	1545° F	+10° F
BSEP2 GE7*	1537° F	1560° F	+23° F
BSEP2 GE13	1535° F	1545° F	+10° F

* GE7 bounds GE8, GE9, and GE10.