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UNITED STAGES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

October 22, 1998

Mr. Garrett D. Edwards Director-Licensing, MC 62A-1 PECO Energy Company Nuclear Group Headquarters Correspondence Control Desk P.O. Box No. 195 Wayne, PA 19087-0195

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SUBJECT: PEACH BOTTOM ATOMIC POWER STATION (PBAPS), UNIT 2, PROPOSED ALTERNATIVE PLAN FOR EXAMINATION OF REACTOR PRESSURE VESSEL SHELL LONGITUDINAL WELDS (TAC NO. MA1740)

Dear Mr. Edwards:

By letter dated April 2, 1998, as supplemented by letter dated August 12, 1998, PECO Energy Company (the licensee) requested the Nuclear Regulatory Commission (NRC) staff's review and approval of an alternate plan for examination of the PBAPS, Unit 2, reactor pressure vessel (RPV) shell welds.

The proposed alternative is requested for the examination of the RPV circumferential shell welds (Section XI Exam Cat. B-A, Item No. B1.11) as required by 10 CFR 50.55a(g)(6)(ii)(A)(2), and the inservice inspection requirements for circumferential welds in the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, 1980 Edition through Winter 1981 Addenda (Table IWB-2500-1, Examination Category B-A, Item No. B1.11) for two (2) operating cycles.

Additionally, the licensee stated that it is unable to meet the 90% volume coverage requirement for examination of each longitudinal weld of the PBAPS, Unit 2, RPV as required by 10 CFR 50.55a(g)(6)(ii)(A)(2). Therefore, the licensee proposes that an alternative examination plan be accepted in lieu of the 90% volume coverage.

The NRC staff has completed the evaluation of the proposed alternative plan for longitudinal weld examinations. The review of the alternative plan for the examination of circumferential welds is in progress and will be addressed in a separate action.

Pursuant to 10 CFR 50.55a(g)(6)(ii)(A)(5), the staff evaluated the licensee's proposed alternative of examining RPV shell longitudinal welds specified in Item B1.12 of Examination Category B-A, "Pressure Retaining Welds in Reactor Vessel," in Table IWB-2500-1 of Subsection IWB of the 1989 Edition of the ASME Code, Section XI and the augmented examination requirements in 10 CFR 50.55a(g)(6)(ii)(A)(2). The NRC staff has determined that, the licensee is unable to completely comply with the requirements, and the licensee's proposed alternative provides

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reasonable assurance of structural integrity and assures that the proposed alternative would provide an acceptable level of quality and safety. Accordingly, the licensee's proposed alternative is authorized pursuant to 10 CFR 50.55a(g)(6)(ii)(A)(5).

The NRC staff's safety evaluation is enclosed.

Sincerely,

/s/

Robert A. Capra, Director Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

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Enclosure: Safety Evalatuion

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Mr. Garrett D. Edwards PECO Energy Company

CC:

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