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October 26] 1998
1940-98-20603

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report 98-013: Local Leak Rate Test Results in Excess of
Technical Specification Limits Due to
Component Wear

Enclosed is Licensee Event Report 98-013. This event did not impact the health and safety of the public.

If any additional information or assistance is required, please contact Ms. Brenda DeMerchant of my staff at 609-971-4642.

Very truly yours,

Michael B. Roche
Vice President and Director
Oyster Creek

MBR/EDe

Enclosure

cc: Oyster Creek NRC Project Manager
Administrator, Region I
Senior Resident Inspector

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO THE INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Oyster Creek Unit 1		DOCKET NUMBER (2) 50 - 219	PAGE (3) 1 of 3
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TITLE (4)
Local Leak Rate Test Results in Excess of Technical Specification Limits Due to Component Wear

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
09	26	98	98	013	00	10	26	98	FACILITY NAME	05000
									FACILITY NAME	05000

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR (Check one or more) (11)	20.2201(b)	20.2203(a)(2)(v)	<input checked="" type="checkbox"/>	50.73(a)(2)(i)	50.73(a)(2)(viii)
POWER LEVEL (10) 100		20.2203(a)(1)	20.2203(a)(3)(i)		50.73(a)(2)(ii)	50.73(a)(2)(x)
		20.2203(a)(2)(i)	20.2203(a)(3)(ii)		50.73(a)(2)(iii)	73.71
		20.2203(a)(2)(ii)	20.2203(a)(4)		50.73(a)(2)(iv)	OTHER
		20.2203(a)(2)(iii)	50.36(c)(1)		50.73(a)(2)(v)	
		20.2203(a)(2)(iv)	50.36(c)(2)		50.73(a)(2)(vii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Paul Crosby	TELEPHONE NUMBER (Include Area Code) 609-971-4895
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE.)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION	MONTH	DAY	YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On September 26, 1998, Local Leak Rate Testing (LLRT) results indicated that Main Steam Isolation Valve NS03B exceeded the Technical Specification leak rate limit of .05(.75)L_a at 35 psig (equivalent to 15.98 SCFH). The leak was quantified as 32.025 SCFH at 35 psig.

The cause of this occurrence was attributed to component wear.

The safety significance of this event is considered minimal as the total penetration leakage would have been limited by Main Steam Isolation Valve NS04B in the same steam header. Maintenance was performed on the valve to restore seat integrity. The as-left local leak rate test, after repairs, resulted in a leak rate of 1.744 SCFH.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

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		98	013	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

DATE OF OCCURRENCE

The condition being reported was discovered on September 26, 1998.

IDENTIFICATION OF OCCURRENCE

Main Steam Isolation Valve (MSIV) NS03B (EHS SB-ISV) exceeded the leak rate criteria specified in Technical Specification 4.5.D.2. This condition is considered to be reportable in accordance with 10 CFR 50.73(a)(2)(i).

CONDITIONS PRIOR TO OCCURRENCE

The plant was in a cold shutdown condition for a refueling outage when this condition was discovered.

DESCRIPTION OF OCCURRENCE

On September 26, 1998, Local Leak Rate Testing (LLRT) results indicated that Main Steam Isolation Valve NS03B exceeded the Technical Specification leak rate limit of .05(.75) L_v at 35 psig (equivalent to 15.98 SCFH). The leak was quantified as 32.025 SCFH at 35 psig.

APPARENT CAUSE OF OCCURRENCE

The root cause of this occurrence was component wear.

This MSIV has been successfully passing LLRT for the past 25 years without needing any maintenance work on the valve internals. The cause of the LLRT failure is normal wear and aging of the internal valve seats. An out-of-round condition was found with the main body seating area.

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

ANALYSIS OF OCCURRENCE AND SAFETY SIGNIFICANCE

THE MSIVs are containment isolation valves designed to minimize coolant loss from the vessel, and the resultant offsite dose, in the event of a main steamline break accident. The design basis loss of coolant accident was evaluated at the primary containment maximum allowable accident leak rate of 1.0% per day at 35 psig. This leak rate provides adequate margin between projected potential offsite dose and 10 CFR 100 guidelines. This leak rate was not exceeded.

The safety significance of this event is considered minimal. The leakage past the MSIV would have been limited by the leak rate of the other MSIV in the same header which met the leak rate acceptance criteria of Technical Specification 4.5.D.2.

CORRECTIVE ACTIONS

The MSIV was inspected internally. The main seat ring was replaced in the valve body. The internals were replaced with a modified poppet design to improve poppet alignment with the main seat. Currently all four of our MSIVs have had this internal modification installed to improve LLRT reliability. The as-left local leak rate test conducted, after repairs, on October 20, 1998, resulted in a leak rate of 1.744 SCFH.

SIMILAR EVENTS

- LER 82-014 LLRT Failure
- LER 82-019 LLRT Results on MSIVs Outside Limits
- LER 82-020 LLRT Results on MSIVs Greater Than Limits
- LER 83-025 Containment Penetration Found Degraded Due to Isolation Valves Actuator/Valve Linkages out of Adjustment
- LER 91-002 Local Leak Rate Test Results in Excess of Limits Due to Valve Degradation
- LER 92-013 Local Leak Rate Test Results in Excess of Limits Due to Valve Degradation
- LER 94-016 Local Leak Rate Test Results in Excess of Technical Specification Limits Due to Component Wear