

ENCLOSURE 3

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKETS 50-325 & 50-324
OPERATING LICENSES DPR-71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
MAIN STACK RADIATION MONITOR

INSTRUCTIONS FOR INCORPORATION

The proposed changes to the Technical Specifications (Appendix A to Operating Licenses DPR-71 and DPR-62) would be incorporated as follows:

UNIT 1

<u>Remove Page</u>	<u>Insert Page</u>
3/4 3-12	3/4 3-12
3/4 3-16	3/4 3-16
3/4 3-18	3/4 3-18
	3/4 3-18a
3/4 3-21	3/4 3-21
	3/4 3-21a
3/4 3-23	3/4 3-23
3/4 3-26	3/4 3-26

UNIT 2

<u>Remove Page</u>	<u>Insert Page</u>
3/4 3-12	3/4 3-12
3/4 3-16	3/4 3-16
3/4 3-16a	3/4 3-16a
3/4 3-18	3/4 3-18
	3/4 3-18a
3/4 3-21a	3/4 3-21a
3/4 3-23	3/4 3-23
3/4 3-26	3/4 3-26

BB10060213 880927
PDR ADOCK 05000324
P PNU

ENCLOSURE 4

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKETS 50-325 & 50-324
OPERATING LICENSES DPR-71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
MAIN STACK RADIATION MONITOR

SUMMARY LIST OF REVISIONS

UNIT 1

<u>Pages</u>	<u>Description of Changes</u>
3/4 3-12	Adds Main Stack Radiation - High and applicable information to Table 3.3.2-1.
3/4 3-16	Adds Action 28 and Note (h) to Table 3.3.2-1.
3/4 3-18	Adds Main Stack Radiation - High and applicable information to Table 3.3.2-2.
3/4 3-18a	Repagination only.
3/4 3-21	Footnote moved "as-is" to Notes portion of Table 3.3.2-2.
3/4 3-21a	Adds Notes section and Note (a) and incorporates footnote (*) from page 3/4 3-21 to Table 3.3.2-2.
3/4 3-23	Adds Main Stack Radiation - High and applicable information to Table 3.3.2-1.
3/4 3-26	Adds Main Stack Radiation - High and applicable information to Table 4.3.2-1.

UNIT 2

<u>Pages</u>	<u>Description of Changes</u>
3/4 3-12	Adds Main Stack Radiation - High and applicable information to Table 3.3.2-1.

3/4 3-16	Adds Action 28 and corrects editorial errors to Table 3.3.2-1.
3/4 3-16a	Adds Note (i), corrects editorial error and repaginates.
3/4 3-18	Adds Main Stack Radiation - High and applicable information to Table 3.3.2-2.
3/4 3-18a	Repagination only.
3/4 3-21a	Adds Note (c) to Table 3.3.2-2.
3/4 3-23	Adds Main Stack Radiation - High and applicable information to Table 3.3.2-3.
3/4 3-26	Adds Main Stack Radiation - High to Table 4.3.2-1.

ENCLOSURE 5

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1
NRC DOCKETS 50-325
OPERATING LICENSES DPR-71
REQUEST FOR LICENSE AMENDMENT
MAIN STACK RADIATION MONITOR

TECHNICAL SPECIFICATION PAGES

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>VALVE GROUPS OPERATED BY SIGNAL(a)</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>				
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010A,B,C,D; B21-TS-N011A,B,C,D; B21-TS-N012A,P,C,D; B21-TS-N013A,B,C,D)	1	2 ^(d)	1, 2, 3	21
e. Condenser Vacuum - Low (B21-PT-N056A,B,C,D) (B21-PTM-N056A-1,B-1,C-1,D-1)	1	2	1, 2 ^(e)	21
f. Turbine Building Area Temperature - High (B21-TS-3225A,B,C,D; B21-TS-3226A,B,C,D; B21-TS-3227A,B,C,D; B21-TS-3228A,B,C,D; B21-TS-3229A,B,C,D; B21-TS-3230A,B,C,D; B21-TS-3231A,B,C,D; B21-TS-3232A,B,C,D)	1	4 ^(d)	1, 2, 3	21
g. Main Stack Radiation - High	(h)	1	1, 2, 3	28

BRUNSWICK - UNIT 1

3/4 3-12

Amendment No.

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATIONACTIONS

- ACTION 20 - Be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- ACTION 21 - Be in at least START-UP with the main steam line isolation valves closed within 2 hours or be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the next 30 hours.
- ACTION 22 - Be in at least START-UP within 2 hours.
- ACTION 23 - Establish SECONDARY CONTAINMENT INTEGRITY with the standby gas treatment system operating within one hour.
- ACTION 24 - Isolate the reactor water cleanup system.
- ACTION 25 - Close the affected system isolation valves and declare the affected system inoperable.
- ACTION 26 - Verify power availability to the bus at least once per 12 hours.
- ACTION 27 - Deactivate the shutdown cooling supply and reactor vessel head spray isolation valves in the closed position until the reactor steam dome pressure is within the specified limits.
- ACTION 28 - Close the affected isolation valves within 14 days or be in HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the following 24 hours.

NOTES

- * When handling irradiated fuel in the secondary containment.
- (a) See Specification 3.6.3.1, Table 3.6.3.1-1 for valves in each valve group.
- (b) A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.
- (c) With only one channel per trip system, an inoperable channel need not be placed in the tripped condition where this would cause the Trip Function to occur. In these cases, the inoperable channel shall be restored to OPERABLE status within 2 hours or the ACTION required by Table 3.3.2-1 for that Trip Function shall be taken.
- (d) A channel is OPERABLE if 2 of 4 instruments in that channel are OPERABLE.
- (e) With reactor steam pressure \geq 500 psig.
- (f) Closes only RWCU outlet isolation valve.
- (g) Alarm only.
- (h) Isolates containment purge and vent valves.

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>		
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010A, B, C, D; B21-TS-N011A, B, C, D; B21-TS N012A, B, C, D; B21-TS-N013A, B, C, D)	$\leq 200^{\circ}\text{F}$	$\leq 200^{\circ}\text{F}$
e. Condenser Vacuum - Low (B21-PTM-N056A-1, B-1, C-1, D-1)	≥ 7 inches Hg vacuum	≥ 7 inches Hg vacuum
f. Turbine Building Area Temp - High (B21-TS-3225A, B, C, D; B21-TS-3226A, B, C, D; B21-TS-3227A, B, C, D; B21-TS-3228A, B, C, D; B21-TS-3229A, B, C, D; B21-TS-3230A, B, C, D; B21-TS-3231A, B, C, D; B21-TS-3232A, B, C, D)	$\leq 200^{\circ}\text{F}$	$\leq 200^{\circ}\text{F}$
g. Main Stack Radiation - High	(a)	(a)

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>		
2. <u>SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Building Exhaust Radiation - High (D12-RM-N010A,B)	≤ 11 mr/hr	≤ 11 mr/hr
b. Drywell Pressure - High (C71-PTM-N002A-1,B-1,C-1,D-1)	≤ 2 psig	≤ 2 psig
c. Reactor Vessel Water Level - Low, Level 2 (B21-LTM-N024A-1,B-1 and B21-LTM-N025A-1,B-1)	$\geq + 112$ inches*	$\geq + 112$ inches*

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
4. RCIC Steam Line Tunnel Temp - High (E51-TS-3319; E51-TS-3320; E51-TS-3321; E51-TS-3322; E51-TS-3323; E51-TS-3355; E51-TS-3487)	$\leq 175^{\circ}\text{F}$	$\leq 175^{\circ}\text{F}$
5. Bus Power Monitor (E51-K42 and E51-K43)	NA	NA
6. RCIC Turbine Exhaust Diaphragm Pressure - High (E51-PS-N012A,B,C,D)	≤ 10 psig	≤ 10 psig
7. RCIC Steam Line Ambient Temp - High (E51-TS-N603A,B)	$\leq 200^{\circ}\text{F}$	$\leq 200^{\circ}\text{F}$
8. RCIC Steam Line Area Δ Temp - High (E51-dTS-N604A,B)	$\leq 50^{\circ}\text{F}$	$\leq 50^{\circ}\text{F}$
9. RCIC Equipment Room Ambient Temp - High (E51-TS-N602A,B)	$\leq 175^{\circ}\text{F}$	$\leq 175^{\circ}\text{F}$
10. RCIC Equipment Room Δ Temp - High (E51-dTS-N601A,B)	$\leq 50^{\circ}\text{F}$	$\leq 50^{\circ}\text{F}$
5. <u>SHUTDOWN COOLING SYSTEM ISOLATION</u>		
a. Reactor Vessel Water Level - Low, Level 1 (B21-LTM-N017A-1,B-1,C-1,D-1)	$\geq + 162.5$ inches*	$\geq + 162.5$ inches*
b. Reactor Steam Dome Pressure - High (B32-PS-N018A,B)	≤ 140 psig	≤ 140 psig

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

NOTES

- (a) Establish alarm/trip setpoints per the methodology contained in the OFFSITE DOSE CALCULATION MANUAL (ODCM).
- * Vessel water levels refer to REFERENCE LEVEL ZERO.

TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>RESPONSE TIME (Seconds)#</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>	
f. Turbine Building Area Temperature - High (B21-TS-3225A,B,C,D; B21-TS-3226A,B,C,D; B21-TS-3227A,B,C,D; B21-TS-3228A,B,C,D; B21-TS-3229A,B,C,D; B21-TS-3230A,B,C,b; B21-TS-3231A,B,C,D; B21-TS-3232A,B,C,D)	NA
g. Main Stack Radiation - High ^(b)	≤ 1.0*
2. <u>SECONDARY CONTAINMENT ISOLATION</u>	
a. Reactor Building Exhaust Radiation - High ^(b) (D21-RM-N010A,B)	≤13
b. Drywell Pressure - High (C71-PT-N002A,B,C,D) (C71-PTM-N002A-1,B-1,C-1,D-1)	≤13
c. Reactor Vessel Water Level - Low, Level 2 (B21-LT-N024A-1,B-1 and B21-LT-N025A-1,B-1) (B21-LTM-N024A-1,B-1 and B21-LTM-N025A-1,B-1)	≤1.0*
3. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>	
a. Δ Flow - High (G31-dFS-N603-1A,1B)	≤13
b. Area Temperature - High (G31-TS-N600A,B,C,D,E,F)	≤13
c. Area Ventilation Temperature ΔT - High (G31-TS-N602A,B,C,D,E,F)	≤13
d. SLCS Initiation (C41A-S1)	NA
e. Reactor Vessel Water Level - Low, Level 2 (B21-LT-N024 A-1,B-1 and B21-LT-N025 A-1,B-1) (B21-LTM-N024 A-1,B-1 and B21-LTM-N025 A-1,B-1)	≤1.0*

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>				
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010A,B,C,D; B21-TS-N011A,B,C,D; B21-TS-N012A,B,C,D; B21-TS-N013A,B,C,D)	NA	M	R	1, 2, 3
e. Condenser Vacuum - Low (B21-PT-N056A,B,C,D) (B21-PTM-N056A-1,B-1,C-1,D-1)	NA ^(a) D	NA M	R ^(b) M	1, 2# 1, 2#
f. Turbine Building Area Temp-High (B21-TS-3225A,B,C,D; B21-TS-3226A,B,C,D; B21-TS-3227A,B,C,D; B21-TS-3228A,B,C,D; B21-TS-3229A,B,C,D; B21-TS-3230A,B,C,D; B21-TS-3231A,B,C,D; B21-TS-3232A,B,C,D)	NA	M	R	1, 2, 3
g. Main Stack Radiation - High	NA	Q	R	1, 2, 3

BRUNSWICK - UNIT 1

3/4 3-26

Amendment No.

ENCLOSURE 6

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2
NRC DOCKETS 50-324
OPERATING LICENSES DPR-62
REQUEST FOR LICENSE AMENDMENT
MAIN STACK RADIATION MONITOR

TECHNICAL SPECIFICATION PAGES

TABLE 3.3.2-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>VALVE GROUPS OPERATED BY SIGNAL(a)</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>				
4. Flow - High (B21-PDTS-N006A-2; B21-PDTS-N007B-2; B21-PDTS-N008C-2; B21-PDTS-N009D-2)	1	2	2, 3	21
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010A, B, C, D; B21-TS-N011A, B, C, D; B21-TS-N012A, B, C, D; B21-TS-N013A, B, C, D)	1	2 ^(d)	1, 2, 3	21
e. Condenser Vacuum - Low (B21-PT-N056A, B, C, D) (B21-PTM-N056A-1, B-1, C-1, D-1)	1	2	1, 2 ^(e)	21
f. Turbine Building Area Temperature - High (B21-TS-3225A, B, C, D; B21-TS-3226A, B, C, D; B21-TS-3227A, B, C, D; B21-TS-3228A, B, C, D; B21-TS-3229A, B, C, D; B21-TS-3230A, B, C, D; B21-TS-3231A, B, C, D; B21-TS-3232A, B, C, D)	1	4 ^(d)	1, 2, 3	21
g. Main Stack Radiation - High	(i)	1	1, 2, 3	28

TABLE 3.3.2-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION

ACTIONS

- ACTION 20 - Be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- ACTION 21 - Be in at least STARTUP with the main steam line isolation valves closed within 2 hours or be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the next 30 hours.
- ACTION 22 - Be in at least STARTUP within 2 hours.
- ACTION 23 - Establish SECONDARY CONTAINMENT INTEGRITY with the standby gas treatment system operating within one hour.
- ACTION 24 - Isolate the reactor water cleanup system.
- ACTION 25 - Close the affected system isolation valves and declare the affected system inoperable.
- ACTION 26 - Verify power availability to the bus at least once per 12 hours.
- ACTION 27 - Deactivate the shutdown cooling supply and reactor vessel head spray isolation valves in the closed position until the reactor steam dome pressure is within the specified limits.
- ACTION 28 - Close the affected isolation valves within 14 days or be in HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the following 24 hours.

NOTES

- * When handling irradiated fuel in the secondary containment.
- (a) See Specification 3.6.3.1, Table 3.6.3.1-1 for valves in each valve group.
- (b) A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.
- (c) With only one channel per trip system, an inoperable channel need not be placed in the tripped condition where this would cause the Trip Function to occur. In these cases, the inoperable channel shall be restored to OPERABLE status within 2 hours or the ACTION required by Table 3.3.2-1 for that Trip Function shall be taken.
- (d) A channel is OPERABLE if 2 of 4 instruments in the channel are OPERABLE.

TABLE 3.3.2-1 (Continued)ISOLATION ACTUATION INSTRUMENTATIONNOTES

- (e) With reactor steam pressure \geq 500 psig.
- (f) Closes only RWCU outlet isolation valve.
- (g) Alarm only.
- (h) Within 24 hours prior to the planned start of the hydrogen injection test, with reactor power at greater than 22% of rated thermal power, the normal full-power radiation background level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip setpoints may be adjusted during the test program based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and the associated trip setpoints shall be set within 24 hours of re-establishing normal radiation levels after completion of the hydrogen injection test or within 12 hours of establishing reactor power levels below 22% of rated thermal power, while these functions are required to be operable.
- (i) Isolates containment purge and vent valves.

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>		
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010A,B,C,D; B21-TS-N011A,B,C,D; B21-TS N012A,B,C,D; B21-TS-N013A,B,C,D)	$\leq 200^{\circ}\text{F}$	$\leq 200^{\circ}\text{F}$
e. Condenser Vacuum - Low (B21-PTM-N056A-1,B-1,C-1,D-1)	≥ 7 inches Hg vacuum	≥ 7 inches Hg vacuum
f. Turbine Building Area Temp - High (B21-TS-3225A,B,C,D; B21-TS-3226A,B,C,D; B21-TS-3227A,B,C,D; B21-TS-3228A,B,C,D; B21-TS-3229A,B,C,D; B21-TS-3230A,B,C,D; B21-TS-3231A,B,C,D; B21-TS-3232A,B,C,D)	$\leq 200^{\circ}\text{F}$	$\leq 200^{\circ}\text{F}$
g. Main Stack Radiation - High	(c)	(c)

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>		
2. <u>SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Building Exhaust Radiation - High (D12-RM-N010A,B)	≤ 11 mr/hr	≤ 11 mr/hr
b. Drywell Pressure - High (C72-PTM-N002A-1,B-1,C-1,D-1)	≤ 2 psig	≤ 2 psig
c. Reactor Vessel Water Level - Low, Level 2 (B21-LTM-N024A-1-1,B-1-1 and B21-LTM-N025A-1-1,B-1-1)	$\geq + 112$ inches ^(a)	$\geq + 112$ inches ^(a)

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTSNOTES

- (a) Vessel water levels refer to REFERENCE LEVEL ZERO.
- (b) Within 24 hours prior to the planned start of the hydrogen injection test, with reactor power at greater than 22% of rated thermal power, the normal full-power radiation background level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip setpoints may be adjusted during the test program based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and the associated trip setpoints shall be set within 24 hours of re-establishing normal radiation levels after completion of the hydrogen injection test or within 12 hours of establishing reactor power levels below 22% of rated thermal power, while these functions are required to be operable.
- (c) Establish alarm/trip setpoints per the methodology contained in the OFFSITE DOSE CALCULATION MANUAL (ODCM).

TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>RESPONSE TIME (Seconds)#</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>	
f. Turbine Building Area Temperature - High (B21-TS-3225A,B,C,D; B21-TS-3226A,B,C,D; B21-TS-3227A,B,C,D; B21-TS-3228A,B,C,D; B21-TS-3229A,B,C,D; B21-TS-3230A,B,C,D; B21-TS-3231A,B,C,D; B21-TS-3232A,B,C,D)	NA
g. Main Stack Radiation - High ^(b)	≤ 1.0*
2. <u>SECONDARY CONTAINMENT ISOLATION</u>	
a. Reactor Building Exhaust Radiation - High ^(b) (D12-RM-N010A,B)	≤ 13
b. Drywell Pressure - High (C72-PT-N002A,B,C,D) (C72-PTM-N002A-1,B-1,C-1,D-1)	≤ 13
c. Reactor Vessel Water Level - Low, Level 2 (B21-LT-N024A-1,B-1 and B21-LT-N025A-1,B-1) (B21-LTM-N024A-1-1,B-1-1 and B21-LTM-N025A-1-1,B-1-1)	≤ 1.0*
3. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>	
a. Δ Flow - High (G31-dFS-N603-1A,1B)	≤ 13
b. Area Temperature - High (G31-TS-N600A,B,C,D,E,F)	≤ 13
c. Area Ventilation Temperature Δ T - High (G31-TS-N602A,B,C,D,E,F)	≤ 13
d. SLCS Initiation (C41A-S1)	NA
e. Reactor Vessel Water Level - Low, Level 2 (B21-LT-N024A-1,B-1 and B21-LT-N025A-1,B-1) (B21-LTM-N024A-1-1,B-1-1 and B21-LTM-N025A-1-1,B-1-1)	≤ 1.0*

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>				
4. Flow - High (B21-PDTS-N006A-2; B21-PDTS-N007B-2; B21-PDTS-N008C-2; B21-PDTS-N009D-2)	D	M	M	2, 3
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010A,B,C,D; 321-TS-N011A,B,C,D; B21-TS-N012A,B,C,D; B21-TS-N013A,B,C,D)	NA	M	R	1, 2, 3
e. Condenser Vacuum - Low (B21-PT-N056A,B,C,D) (B21-PTM-N056A-1,R-1,C-1,D-1)	NA ^(a) D	NA M	R ^(b) M	1, 2# 1, 2#
f. Turbine Building Area Temp-High (B21-TS-3225A,B,C,D; B21-TS-3226A,B,C,D; B21-TS-3227A,B,C,D; B21-TS-3228A,B,C,D; B21-TS-3229A,B,C,D; B21-TS-3230A,B,C,D; B21-TS-3231A,B,C,D; B21-TS-3232A,B,C,D)	NA	M	R	1, 2, 3
g. Main Stack Radiation - High	NA	Q	R	1, 2, 3

BRUNSWICK - UNIT 2

3/4 3-26

Amendment No.