Louisiana Power and Light Company
Waterford 3 SES
Docket No. 50-382, License No. NPF-38

REPORT OF FACILITY CHANGES, TESTS AND EXPERIMENTS

FOR 1985 PER 10CFR50.59

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Louisiana Power & Light Company
Waterford 3 SES
10CFR50.59 ANNUAL REPORT FOR 1985

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Waterford 3 SES
1985 Report of Facility Changes, Tests and Experiments

SUMMARY

This report provides the Waterford 3 Facility Changes and Tests made pursuant to 10CFR50.59(a)(1). The report covers the period from December 18, 1984 (issuance of low power operating license NPF-26) through December 31, 1985.

No experiments were conducted at Waterford 3 during the report period. None of the items in this report represents an unreviewed safety question or a change to the Technical Specifications.

The report identifies 49 Facility Changes (46 changes under the Station Modification program, two procedure changes and one special safety evaluation change to the FSAR), and 4 line item Test Changes which includes 8 specific changes to the Initial Test Program previously reported to NRC in fulfillment of License NPF-38 Condition 2.C.10.

I. FACILITY CHANGES

A. Station Modifications (SMPs)

1. Station Modification, SMP-0004

SMP-0004, Addition of Shunts to Battery Disconnect Switches 3A-DC-S and 3B-DC-S

Description of Change

Installed new shunts in the 3A-DC-S and 3B-DC-S battery disconnect switches and connected them with the battery ammeters.

Reason for Change

In order to provide proper ammeter indication, the original shunts had to be disconnected and new shunts added directly to the battery leads.

Safety Evaluation

The new shunts are being added to a Class 1E, seismic category I system. The shunts are seismically supported and their installation is verified to meet the requirements of IEEE-344. Addition of shunts to the battery disconnect switches does not affect any descriptions in the FSAR. There is no increase in the possibility of an accident and no accident scenarios not previously considered have been created.

2. Station Modification, SMP-0025

SMP-0025, Transfer of Loads to PDP3014AB

Description of Change

This modification provided for the redistribution of loads from Static Uninterruptible Power Supply 3AB to 3014AB.

Reason for Change

To alleviate the problem of overloading on the 3AB inverter.

Safety Evaluation

The reassignments of the loads from the panel are considered as non-safety. These changes do not increase the possibility of an accident or create any new accident scenarios not previously considered.

3. Station Modification, SMP-0035

SMP-0035, Replace Bypass Transformers for Elgar Inverters

Description of Change

This modification provided the engineering information necessary to replace the bypass power supplies to Class 1E Static Uninterruptible Power Supplies (SUPS) 3A-S and 3B-S.

Reason for Change

The bypass transformers for Elgar SUPS did not provide voltage regulation or transient suppression within the specified limits.

Safety Evaluation

The changes do not adversely affect operation of the equipment. There is no increase in the possibility or probability of an accident or malfunction.

4. Station Modification, SMP-0090

SMP-0090, Tie-In for Condensate Polisher

Description of Change

Provided design and installation of piping for condensate polisher tie-in to condensate system.

Reason for Change

The condensate polishers were added to the condensate system to provide a positive means of chemistry control.

Safety Evaluation

The condensate polishers are non-safety. The addition of this system does not impact the safe operation or shutdown of the plant. This change does not increase the possibility or probability of any accident or malfunction previously evaluated.

5. Station Modification, SMP-0094

SMP-0094, Pressurizer Heater Time Delay Energization

Description of Change

This change provided the means for reclosing the station service transformers feeder breakers so that the proportional pressurizer heaters can be loaded onto the emergency bus during SIAS presence and offsite power available.

Reason for Change

To permit the operator to remotely energize the pressurizer heater to the emergency bus during an emergency.

Safety Evaluation

This change does not affect the isolation criteria as set forth in the FSAR. This change does not increase the possibility or probability of any accident or malfunction previously evaluated.

6. Station Modification, SMP-0122

SMP-0122, Boron Management System Discharge Cross Tie

Description of Change

This modification provides the design engineering required to cross tie the Waste Management System effluent discharge with the Boron Management System discharge to the Circulating Water System.

Reason for Change

To provide tie-in connections to allow waste management system effluent to be discharged through the boron management system flowpath.

Safety Evaluation

The system is classified as non-safety and non-seismic. The addition of two flanged tie-in connections does not increase the possibility of an accident or create any new accident scenarios not previously considered.

7. Station Modification, SMP-0127

SMP-0127, Piping Connections for New Auxiliary Feedwater Pump

Description of Change

This change provided the piping and electrical tie-in connections required to implement the new Auxiliary Feedwater System.

Reason for Change

To make the Auxiliary Feedwater System functional.

Safety Evaluation

The system is classified as non-safety and non-seismic. The addition of the system tie-ins does not increase the possibility of an accident or create any accident scenarios not previously considered.

8. Station Modification, SMP-0128

SMP-0128, Station Air and Instrument Air Load/Unload Pressure Switches

Description of Change

Replace load/unload pressure switches with switches having higher setpoints.

Reason for Change

To maintain N_2 accumulator manifold pressure at or above 92 psig during normal operation.

Safety Evaluation

These pressure switches are non-safety. No new accidents or malfunctions are created with the change and no previously evaluated malfunction is increased.

9. Station Modification, SMP-0167

SMP-0167, Safety Injection Flow Diagram Corrections

Description of Change

Safety Injection System flow diagram, LOU1564-G-167 sheet 1, has minor discrepancies between the flow diagram and the field. One discrepancy is a typographical error and the other is a valve in different location.

Reason for Change

To change plant drawings to reflect as-built conditions.

Safety Evaluation

The correction of the flow diagram does not effect the probabilities or consequences of an accident or equipment malfunction.

10. Station Modification, SMP-0176

SMP-0176, Decontamination Room and Hot Machine Shop Drain Piping Vent

Description of Change

This modification is to complete the tie-in within the pipe chase for the vent line from decontamination room and hot machine shop drain piping to the vent gas collection header.

Reason for Change

To permit discharge to the vent gas collection header from the Decontamination Room and the Hot Machine Shop.

Safety Evaluation

The system is classified as non-safety, non-seismic. The completion of the tie-in between the 2" vent line and vent gas collection header within the pipe chase does not increase the possibility of an accident or create any new accident scenarios not previously considered.

11. Station Modification, SMP-0192

SMP-0192, Resolution of Appendix R Deviations

Description of Change

This modification added emergency 8-hour battery packs, documented as-built condition of existing conduits, rerouted certain essential cables, and provided fire-wrap for containment cable tray.

Reason for Change

To revise plant drawings to reflect as-built conditions and to provide for miscellaneous cable wrapping and additional emergency lighting.

Safety Evaluation

The changes made by this modification do not adversely affect plant operation or equipment. There is no increase in the possibility or probability of an accident or malfunction previously evaluated.

12. Station Modification, SMP-0210

SMP-0210, Demineralized Water Lines

Description of Change

Change flow diagram for several demineralized water lines to reflect as-built configuration of the demineralized water system.

Reason for Change

This modification is to correct a flow diagram to show the as-built configuration of several demineralized water lines.

Safety Evaluation

The Demineralized Water System serves no safety function and is not required to achieve safe shutdown or mitigate the consequences of an accident. This change does not increase the possbility or probability of any accident or malfunction previously evaluated.

13. Station Modification, SMP-0211

SMP-0211, Revise Line and Valve List for Correct Design/Hydrostatic Pressure for Line 3FW2-42 A/B.

Description of Change

Updating of line and valve list.

Reason for Change

The Balance of Plant Line List and Valve List was updated to indicate correct design and hydrotest pressure for a feedwater line and valve. A flow diagram was revised to reflect the actual physical sequence of connections for another feedwater line.

Safety Evaluation

This modification does not change the function of the equipment. The probabilities or consequences of an accident or equipment malfunction are not effected.

14. Station Modification, SMP-0213

SMP-0213, Condensate Pump Seal Piping

Description of Change

This modification changed the flow diagram to reflect the actual sequence of the condensate pump seal water lines from the supply header.

Reason for Change

To change plant drawings to reflect as-built conditions.

Safety Evaluation

The Condensate System is non-safety related; and non-seismic. This change does not increase the possibility o. probability of accidents previously evaluated.

15. Station Modification, SMP-0220

SMP-0220, Turbine Lube Oil System

Description of Change

Update flow diagram to indicate the correct line sizes and to indicate non-vendor furnished lines on the oil vapor extractor drain.

Reason for Change

To add line numbers and valve numbers to the line list and the flow diagram.

Safety Evaluation

The Oil Vapor Extractor Drain Lines are classified as non-safety, non-seismic and do not perform any safety-related function. This change does not increase the possibility or probability of any accident or equipment malfunction previously evaluated.

16. Station Modification, SMP-0244

SMP-0244, Reactor Trip on Turbine Trip

Description of Change

This modification provided the means to trip the reactor on turbine trip signal when the Reactor Power Cutback System is inoperable.

Reason for Change

To provide the backup protection afforded by the non-safety reactor trip on turbine trip.

Safety Evaluation

This modification and test are supplemental and are provided as a backup to the Reactor Power Cutback System. The system is designed to reduce reactor power prior to safety systems being activated. This change does not increase the possibility or probability of any accident or malfunction previously evaluated.

17. Station Modification, SMP-0266

SMP-0266, Blowdown Heat Exchanger Tubeside Relief Valves

Description of Change

To revise Flow Diagram to show the existing relief valves on the Circulating Water side of the Blowdown Heat Exchanger.

Reason for Change

To revise plant drawings to reflect as-built conditions.

Safety Evaluation

This modification was a paper change only to show vendor supplied relief valves which are classified non-safety, non-seismic, and does not affect previously performed accident analyses.

18. Station Modification, SMP-0326

SMP-0326, Computer Power Supply Battery

Description of Change

Replace all Computer SUPS Battery Cells with upgraded cells.

Reason for Change

The computer SUPS battery bank had 20 permanently damaged cells and the remaining 160 cells were warped.

Safety Evaluation

The battery bank is a non-1E system and is not needed to assure safe-shutdown of the plant. Also, the new batteries will provide for more computer time without loss of power than the old battery system. This change does not increase the possibility or probability of any accident or malfunction previously evaluated.

19. Station Modification, SMP-0354

SMP-0354, Additional Cooler Capacity and Dissolved Oxygen Analyzer

Description of Change

This modification is for the addition of a second sample cooler to be cooled by component cooling water and a new dissolved oxygen analyzer.

Reason for Change

To provide the capability to sample and analyze in the three-hour time limit which is in accordance with NUREG-0737.

Safety Evaluation

The Post Accident Sampling System (PASS) is classified as non-safety and non-seismic. However, a portion of the component cooling water piping servicing the additional sample cooler is safety Class 3 and Seismic Category I.

The addition of the miscellaneous system tie-in connections and the replacement of the dissolved oxygen analyzer do not increase the possibility of an accident or create any new accident scenarios not previously considered.

20. Station Modification, SMP-0366

SMP-0366, Condensate System Flow Diagram

Description of Change

This modification provided for updating condensate system flow diagrams.

Reason for Change

To reflect an actual line location and corrected drawing reference coordinates.

Safety Evaluation

This portion of the condensate system is designated non-safety, non-seismic and is not required to mitigate the consequences of an accident. This change does not increase the possibility or probability of any accident or malfunction previously evaluated.

21. Station Modification, SMP-0371

SMP-0371, Update Dwg. G-162 Sh. 3 per PAS As-Built Condition

Description of Change

This modification provided for updating the post accident sampling system flow diagram.

Reason for Change

Drawing G-162, Sh. 3, is updated to reflect the as-built condition.

Safety Evaluation

This change does not increase the possibility, probability, or consequences of any accident previously evaluated in the FSAR.

22. Station Modification, SMP-0417

SMP-0417, Reactor Auxiliary Building Operator Enclosure

Description of Change

An 11 ft. by 6 ft. enclosure was installed on the -4 foot elevation for auxiliary operator use.

Reason for Change

To provide break and eating area in the immediate area of the RCA access point.

Safety Evaluation

This modification has no impact on any systems required for safe shutdown of the plant, nor does it have any impact on accidents previously evaluated in the FSAR.

23. Station Modification, SMP-0485

SMP-0485, Additional Telephone in CAS

Description of Change

An additional telephone was installed in the Central Alarm Station room for onsite use only.

Reason for Change

The existing telephone in the CAS did not provide sufficient telephone communication capability for emergencies.

Safety Evaluation

The modification is in the non-safety related, non-seismic Communications System. The Communications System is not required for safe shutdown of the plant and the modification has no impact on accidents previously evaluated in the FSAR.

24. Station Modification, SMP-0514

SMP-0514, Health Physics Control Point Office Modification

Description of Change

Modify HP control point office by adding two doors, a sliding window and a service counter.

Reason for Change

To improve flow of traffic into the Radiation Controlled Area (RCA).

Safety Evaluation

The system is classified as non-safety and non-seismic. The addition of two doors and one window does not increase the possibility of an accident or create a new accident scenario not previously considered.

25. Station Modification, SMP-0531

SMP-0531, Steam Binding of EFW Pumps (IEN 84-06)

Description of Change

Install local temperature indicators on discharge side of EFW pumps. The instrumentation will provide indication if the line heats up and there is a possibility for steam forming in the line.

Reason for Change

To implement recommendations resulting from corrective actions required to resolve IEN 84-06.

Safety Evaluation

The changes provided with this modification will not adversely affect operation of the EFW piping system. The purpose of the changes is to further reduce the possibility of an accident or malfunction.

26. Station Modification, SMP-0597

SMP-0597, Addition of CPU-4 To Plant Monitoring Computer

Description of Change

Addition of a fourth CPU System to the existing three CPUs of the Plant Monitoring Computer.

Reason for Change

The original system did not allow for performance of all functions in real time under full loading conditions.

Safety Evaluation

Installation of the fourth CPU system will not affect plant operation due to the dual complex - redundant Host Computer System. This change will not increase the possibility or probability of any accident or malfunction previously evaluated.

27. Station Modification, SMP-0630

SMP-0630, Plant Protection System Setpoint/Plant Monitoring Computer Interface

Description of Change

This modification removed the Plant Protection System setpoint/Plant Monitoring Computer cables.

Reason for Change

To relieve concerns that Plant Monitoring Computer Multiplexer site component failures would effect Plant Protection System input signals.

Safety Evaluation

Deletion of the setpoint computer points does not increase the possibility or probability of any accident or malfunction previously evaluated.

28. Station Modification, SMP-0685

SMP-0685, Seismic Rack for Scaffold Storage inside Reactor Containment Building

Description of Change

This modification covered design and installation of a seismically qualified storage rack to hold scaffolding that has been painted and designated for use in the reactor containment building (RCB).

Reason for Change

To provide adequate storage for scaffolding in RCB thereby eliminating need to remove all scaffolding after each work effort in containment.

Safety Evaluation

The system is classified as non-safety and seismically designed to withstand safe shutdown earthquake. The addition of the storage rack does not increase the possibility of an accident or create any new accident scenario not previously considered. The hydrogen generation due to galvanized clamps has been studied and it was concluded that the small amount of galvanized coating on these clamps is not considered detrimental with regard to the quantity of hydrogen gas produced during a LOCA. No aluminum material has been added.

29. Station Modification, SMP-0730

SMP-0730, Relief Valve to Replace Rupture Disc.

Description of Change

This station modification changed the Condensate Hotwell dump lines 6CD6-55 and 6CD6-252 by adding a multiple stage orifice plate sized to reduce system pressure from Condensate Pump discharge pressure to 50psig at a flow rate of approximately 1400gpm. This change also added a relief valve to replace the existing rupture disc.

Reason for Change

This change replaced restriction orifice 6CD-K162 with a multiple stage orifice designed to eliminate cavitation which causes vibration and erosion.

Safety Evaluation

The modifications are in the non-safety related Condensate System located in the Turbine Building and the yard south of the Turbine Building. The Condensate System is not required for safe shutdown of the plant and the modifications have no impact on accidents previously evaluated in the FSAR. The portions of the system affected by this SMP are only used during start-up as the system water chemistry is being adjusted and their failure in that mode would not have any adverse radiological consequences. The existing rupture disc was replaced with a relief valve with a setpoint of 100psig. This replacement does not introduce any new flow paths which have not been analyzed before.

30. Station Modification, SMP-0747

SMP-0747, Fuel Handling Building HVAC Flappers

Description of Change

Addition of restictions at core holes on the -35 elevation of the Fuel Handling Building.

Reason for Change

To provide a positive means for maintaining a negative pressure in the Fuel Handling Building.

Safety Evaluation

The walls where flappers are to be installed are seismic Category I. This modification does not increase the probability of any previously evaluated accident and equipment malfunction, and it does not create the possibility of any new accident or equipment malfunction. The margin of safety defined in any technical specification is not reduced.

31. Station Modification, SMP-0760

SMP-0760, Reactor Coolant Pump Seal Water Circuit Modification

Description of Change

Revise seal cooler isolation valve circuitry to allow reopening the valves from the control room.

Control circuit was revised to have the seal cooler isolation valves controlled by the temperature. If these isolation valves close by the temperature signal, operator can override the temperature signal and reopen the valves by a control switch being added on RTG Board CP-2 Panel.

These valves will open for a preset time and after the lapse of this time, if the high temperature still exists, the valves will reclose. Otherwise they stay open.

Reason for Change

To change seal cooler isolation valve actuation from pressure initiation to temperature initiation. Also, the operator can remotely reopen the valves.

Safety Evaluation

The Component Cooling Water (CCW) to the RCP seal water is a non-essential, seismically qualified system. This change will not increase the possibility or probability of any accident or malfunction previously evaluated.

32. Station Modification, SMP-0765

SMP-0765, Abandon Smoke Detector 31-1

Description of Change

Delete one smoke detector from Fire Zone RAB 31 in the Reactor Auxiliary Building.

Reason for Change

The detector was located in a pipe chase that was inaccessible during plant operations.

Safety Evaluation

The changes will not adversely affect operation of plant. Based on the Fire Hazard Analysis, the creation of the new Fire Zone 23A will not have an adverse impact. An exemption request to delete the specific detector was filed with NRC via letter W3P85-0560, dated March 1, 1985.

33. Station Modification, SMP-0784

SMP-0784, New Main Access Point to the Plant Site

Description of Change

Reduce plant access to a single point for controlling personnel and traffic.

Reason for Change

To reduce the number of entrances to the Owner Controlled Area (OCA) to one.

Safety Evaluation

This change will not increase either t..e probability or the possibility of any accidents previously evaluated. The change will restrict access to the OCA which now includes all parking lots. The complete control of parking lots was not previously possible.

34. Station Modification, SMP-0810

SMP-0810, Power Service and HVAC Duct in -4 elevation HP Area

Description of Change

Provide for the installation of a ventilation duct and electrical power for the Perkin-Elmer 5000 AA Unit.

Reason for Change

The previous equipment was not capable of reaching the level of sensitivity required for iron, copper and sodium in the plant feedwater.

Safety Evaluation

This is a non-safety related modification. A failure of the unit in no way prevents the reactor from shutting down nor can it impact the ability to maintain the plant in a safe shutdown condition. This change does not increase the possibility or probability of any accident or malfunction previously evaluated.

35. Station Modification, SMP-0867

SMP-0867, Condensate Recirculation Lines 6CD6 -113 and 114 Add Orifices

Description of Change

Orifices were added downstream of valves CD137B and CD137C.

Reason for Change

The valves were of the wrong design to effect the necessary pressure drop to achieve design flow of 2800gpm through lines 6CD6 -113 and 114.

Safety Evaluation

The modification is in the non-safety related Condensate System. The Condensate System is not required for safe shutdown of the plant. This change does not increase the possibility or probability of any accident or malfunction previously evaluated.

36. Station Modification, SMP-0875

SMP-0875, Oil Drain to Waste Drain Modification

Description of Change

To provide design and construction information for rerouting of the Charging Pumps and Oil Separator Drains from the Oil Sump of the Liquid Radwaste System.

Reason for Change

The purpose of the modification was to eliminate the possibility of releasing contaminated oil to an unrestricted area.

Safety Evaluation

The changes will not adversely affect operation of the subject piping system. There is no increase in the possibility or probability of an accident or malfunction that was previously evaluated.

37. Station Modification, SMP-0891

SMP-0891, Change Drawing for VCT Sample Point G-9

Description of Change

Change drawing to reflect as-built conditions.

Reason for Change

VCT sample point mislabled as P-9 and needed to be changed to G-9.

Safety Evaluation

This change does not reflect an actual change to the plant and does not reflect a difference from conditions previously evaluated. This change does not increase the possibility or probability or any accident or malfunction previously evaluated.

38. Station Modification, SMP-0896

SMP-0896, CCW Surge Tank Vent Isolation

Description of Change

A one inch check valve, designed for gas service, was installed in the Component Cooling Water Surge Tank Vent line (7CC1-262).

Reason for Change

To prevent activity from the Vent Gas Collection Header from entering the surge tank.

Safety Evaluation

The change involves non-safety equipment and is not required for safe shutdown of the plant. This change does not increase the possibility or probability of any accidents or malfunction previously evaluated.

39. Station Modification, SMP-0934

SMP-0934, CVC-103 Replacement

Description of Change

This modification provided replacement of the WKM supplied CVCS letdown containment isolation valve with a Masoneilan globe valve. It also provided clearance for the operator of the replacement valve by reorienting the hydrotest connection furnished and the replacement valve operator with the necessary tie-ins from electrical and instrument air supplies.

Reason for Change

The original valve consistently stuck in the open position during and after the process of plant cooldown.

Safety Evaluation

The modified portion of the system (with the exception of the leakoff piping and the instrument air supply to the operator) is classified as seismic Category I and safety related. None of the modifications increase the possibility of an accident nor create any new accident scenarios not previously considered.

40. Station Modification, SMP-0964

SMP-0964, Reconnection of Safety MOV Heater Circuit with Double Isolation Device in Containment

Description of Change

This modification provides double circuit protection to meet isolation requirements for 1E and non-1E circuits.

Reason for Change

To assure the non-safety heaters in containment will not affect the 1E power circuits.

Safety Evaluation

The changes will not adversely affect operation of the subject equipment and satisfies the isolation criteria for feeding non-safety systems from 1E power circuits. The change will not increase the probability or possibility of any accident or malfunction previously evaluated.

41. Station Modification, SMP-0974

SMP-0974, Installation of Dual Tower Hydrogen Gas Dryer

Description of Change

Replaced the single tower hydrogen gas dryer with a dual tower unit.

Reason for Change

The performance of the original single tower hydrogen gas dryer was unacceptable.

The replacement of the existing generator hydrogen gas dryer improves generator protection against moisture and reduces future maintenance costs.

Safety Evaluation

The hydrogen gas generator coolant system is non-safety related and non-seismic and is not required to shut down the turbine generator after a turbine trip. This change will not increase the possibility or probability of any accident or malfunction previously evaluated.

42. Station Modification, SMP-1013

SMP-1013, Emergency Diesel Generator "A" Starting Air Compressor Feeder Circuit Downgrade

Description of Change

Add double protection devices in some 480V power circuits.

Reason for Change

To meet separation criteria for non-safety systems that are fed from 1E power circuits.

Safety Evaluation

Single failure of a protective device will not effect the load on the respective 1E power circuit and will provide greater assurance that the separation criteria are being met. This change will not increase the possibility or probability of any accident or malfunction previously evaluated.

43. Station Modification, SMP-1028

SMP-1028, Circulating Water Pumps Bearing Water Supply Line Updates

Description of Change

Treated Water System valves TW-2102, TW-2101 and temporary strainer 7FP-S682 were not in the field but appeared on all plant drawings. These items were not required as other upstream block valves can be used in lieu of TW-2102. The temporary strainer was not necessary as there are individual strainers going to all bearings. All affected drawings and valve lists will be modified.

Reason for Change

To remove two valves and a temporary strainer from plant drawings to reflect as-built conditions.

Safety Evaluation

The modification is in the non-safety related Treated Water System. The Treated Water System is not required for safe shutdown of the plant. This change does not increase the possibility or probability of any accident or malfunction previously evaluated.

44. Station Modification, SMP-1060

SMP-1060, Diesel Generators (A&B) As-Built Conditions

Description of Change

Provides an update on Diesel Generator paperwork to reflect as-built conditions.

Reason for Change

To reflect as-built conditions.

Safety Evaluation

The changes incurred do not affect the operation of the equipment or the safety of the plant. There is no increase in the possibility or the probability of any accidents previously evaluated.

45. Station Modification, SMP-1123

SMP-1123, Polisher Building Sump Return to Backwash

Description of Change

Change valve lineup so that polisher building sump is discharged to backwash tank instead of going to yard oil separator.

Reason for Change

To eliminate discharge of resinous waste to yard oil separator.

Safety Evaluation

This is a non-safety, non-quality system. This change does not increase the possibility or probability of any accidents or malfunction previously evaluated.

46. Station Modification, SMP-1160

SMP-1160, Gland Seal Steam Leak Off Line

Description of Change

Provide additional pressure taps in gland seal steam leakoff lines.

Reason for Change

To eliminate drainage problems caused by slope problems in the lines. The modification enhances operation.

Safety Evaluation

This is a non-safety related system. This change will not increase either the possibility or the probability of any accident previously evaluated.

B. Procedure Changes

47. Procedure, OP-901-056

OP-901-056, Isolation Panel Fire

Description of Change

This was a new procedure to describe the specific actions to be taken in the event of a fire in the +35 elevation RAB Isolation Panel.

Reason for Change

To satisfy 10CFR50, Appendix R commitments to resolve fire watch requirements.

Safety Evaluation

Use of this procedure will not increase the probability or possibility of any accident or malfunction previously evaluated. The bases for this evaluation is documented in the Associated Circuits Analysis - Engineering Report, that was transmitted to the NRC per letter LW3-024-85 dated January 15, 1985.

48. Procedure, HP-1-235, Rev. 3

HP-1-235, Rev. 3, Calculation and Adjustment of Radiation Monitoring Setpoints

Description of Change

This change increased the iodine channel setpoints from $1x10^{-9}$ uCi/ml to $1x10^{-8}$ uCi/ml.

Reason for Change

The orignal setpoint was near the extreme limit of the monitors' sensitivity and would have resulted in frequent nusiance alarms.

Safety Evaluation

The monitors are classified as non-safety and non-seismic. The monitors are not utilized in the accident analysis nor do they have an impact on the accident analysis. This change will not increase the possibility or probability of any accident or malfunction previously evaluated.

C. Special Evaluations

49. Manual Actuation of Reactor Power Cutback System

Description of Change

Activate the Reactor Power Cutback System (RPCS) using the manual rod group select mode.

Reason for Change

The change was needed to allow the operators to preselect rod groups to be inserted in the event of a feedwater pump trip. With the change in effect, the RPCS may be able to reduce power quickly enough to eliminate a reactor trip on low steam generator level.

Safety Evaluation

Based on the fact that the impact of the Reactor Power Cutback System's operation on safety has been previously analyzed, the 10CFR50.59 evaluation revealed no unreviewed safety question. CE report CEN-200 provides the safety analysis to address the implications of the RPCS on reactor safety and found that the plant is protected against single failures of the RPCS. This report was reviewed and found acceptable by the staff in SER Supplement No. 5 (paragraph 4.4.2.2). No scenario of RPCS operation or misoperation can be postulated which would reduce reactor safety or hinder safe shutdown.

II. TEST CHANGES

50. Biological Shield Effectiveness Survey, SIT-TP-715

Description of Change

Change the FSAR to reflect as-found radiation levels in containment and RCA during power ascension testing.

Reason for Change

During the conduct of SIT-TP-715, three areas were found to have radiation levels higher than predicted. None of the radiation levels warranted design changes, therefore the changes will be made to the FSAR.

Safety Evaluation

None of the higher radiation levels represent an unreviewed safety question and do not affect any technical specification safety margin. Operation of the plant with slightly higher than anticipated radiation levels will not have any impact on FSAR accident analysis or other accident possibilities.

51. Startup Test, SIT-TP-735

SIT-TP-735, Incore Dectector Signal Verification

Description of Change

Accpetance of test deficiency for SIT-TP-735.

Reason for Change

Six incore detectors did not meet acceptance criteria as set forth in the procedure.

Safety Evaluation

This deviation does not increase the possibility or probability of any accidents or malfunction previously evaluated. The incore detectors are not required for plant safety since they do not initiate any direct safety-related functions.

52. Startup Integrated Test, SIT-TP-743

SIT-TP-743, Power Ascension Testing Ventilation Capability

Description of Change

Containment ambient air and reactor cavity temperature acceptance criteria were changed to reflect as-found condition.

Reason for Change

During power ascension testing (100% Power), the containment ambient air and reactor cavity temperature exceeded acceptance criteria by 2.8°F and 1.8°F respectively.

Safety Evaluation

The two temperatures being above 120°F will not increase the probability of occurrence of any accident previously calculated nor will it create possibility for an accident or malfunction of a different type than any previously evaluated in the FSAR. No Technical Specification limits will be exceeded as a result of this change.

53. Initial Test Program Changes (FSAR Chapter 14)

Safety evaluations pursuant to 10CFR50.59 were conducted for the test changes described below. These test changes were reported to the NRC in accordance with License NPF-38, Condition 2.C.10, and are included herewith for completeness of the report relative to 10CFR50.59(b). None of the changes to the Initial Test Program required a change to the Technical Specifications or constituted an unreviewed safety question. The required FSAR changes are to be made in accordance with 10CFR50.71(e).

The Phase III - Initial Startup Testing changes included:

(1) FSAR Test 14.2.12.3.31, Control Systems Checkout

Change: The Megawatt Demand Setter (MDS) was deleted as a test prerequisite because Waterford 3 does not have an MDS. Boron concentration was deleted from accpetance criteria since it is maintained manually not by a control system.

Reference Document: Reported to NRC by LP&L letter W3P85-1263, dated 5/10/85.

(2) FSAR Test 14.2.12.3.31, Control Systems Checkout

Change: The 20% reactor power level test was deleted from the test schedule.

Reference Document: Reported to NRC by LP&L letter W3P85-1290, dated 5/17/85.

(3) FSAR Test 14.2.12.3.38, Loss of Load Transients at 50, 80 and 100 Percent Power

Change: Clarification was provided of when the test was to be performed, i.e., loss of load transients will be performed only if the Reactor Power Cutback System is to be declared operational.

Reference Document: Reported to NRC by LP&L letter W3P85-1418, dated 7/8/85.

(4) FSAR Test 14.2.12.3.37, Turbine Trip

Change: Clarification of when the test was to be performed, i.e., to allow the turbine trip test to be conducted at a lower power level.

Reference Document: Reported to NRC by LP&L letter W3P85-1431, dated 7/29/85.

53. (Continued) - Initial Test Program Changes (FSAR Chapter 14)

(5) FSAR Test 14.2.12.3.17, Piping Thermal Growth, Vibration, and Shock

Change: A satisfactory evaluation of piping stresses in main steam piping after the revised test will be acceptable instead of an actual instrumented level 4 transient monitoring of the main steam system during turbine trip from 100% power.

Reference Document: Reported to NRC by LP&L letter W3P85-1441, dated 7/29/85.

(6) FSAR Test 14.2.12.3.31, Control Systems Checkout

Change: Deletion of step and ramp unit load changes at 50% power. The control systems described in the test are not safety related and no credit for their operation is taken in the Waterford 3 FSAR Chapter 15 accident analyses.

Reference Document: Reported to NRC by LP&L letter W3P85-1432, dated 7/29/85.

(7) FSAR Test 14.2.12.3.31, Control Systems Checkout

Change: Deleted the Control Systems steady-state operation checkout at 100% power level.

Reference Document: Reported to NRC by LP&L letter W3P85-3209, dated 10/15/85.

(8) FSAR Test 14.2.12.2.58, Movable Incore Detector System

Change: Deleted the Movable Incore Detector responsiveness testing from the power ascension test program. The system is not required to support any safety analyses.

Reference Document: Reported to NRC by LP&L letter W3P85-3209, dated 10/15/85.

LOUISIANA POWER & LIGHT / 142 DELARONDE STREET
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MIDDLE SOUTH UTILITIES SYSTEM

April 1, 1986

W3P86-0047
A4.05
QA

Mr. Robert D. Martin
Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
1985 Report of Facility Changes, Tests and Experiments

Dear Mr. Martin:

Enclosed is the 1985 Report of Facility Changes, Tests and Experiments for Waterford 3 which is furnished pursuant to 10CFR50.59(a)(1). The annual report covers the period from December 18, 1984 (issuance of low power operating license NPF-26) through December 31, 1985. The original and 39 copies of the report are provided the accordance with 10CFR50.59(b).

If you have any questions regarding this report, please contact G.E. Wuller, Onsite Licensing, at (504) 464-3499.

Very truly yours,

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:ssf

Attachment

cc: NRC, Director, Office of I&E
G.W. Knighton, NRC-NRR
J.H. Wilson, NRC-NRR
NRC Resident Inspectors Office
B.W. Churchill
W.M. Stevenson

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