

October 23, 1998

Mr. Martin L. Bowling, Jr.
Recovery Officer - Technical Services
Northeast Nuclear Energy Company
c/o Ms. Patricia A. Loftus
Director - Regulatory Affairs
P. O. Box 128
Waterford, Connecticut 06385

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT 3 - REQUEST FOR
ADDITIONAL INFORMATION (TAC NO. MA3372)

Dear Mr. Bowling:

In a June 16, 1998, letter, Sargent & Lundy documented two discrepancy reports, DR-MP3-0514 and 0580, which raised issues related to the adequacy of the licensing basis of Millstone Unit 3, rather than Northeast Utilities' compliance with the Millstone Unit 3 licensing basis. In order for the staff to complete its review of the issues, additional information is required.

The enclosed request was discussed with Mr. David Dodson of your staff on October 16, 1998. A mutually agreeable target date of November 30, 1998, for your response was established. If circumstances result in the need to revise the target date, please call me at the earliest opportunity.

Sincerely,

Original signed by D. McDonald for:

James W. Andersen, Project Manager
Millstone Project Directorate
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosure: Request for Additional
Information

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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ADDITIONAL INFORMATION (TAC NO. MA3372)

Dear Mr. Bowling:

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The enclosed request was discussed with Mr. David Dodson of your staff on October 16, 1998. A mutually agreeable target date of November 30, 1998, for your response was established. If circumstances result in the need to revise the target date, please call me at the earliest opportunity.

Sincerely,

A handwritten signature in black ink, appearing to read "James W. Andersen".

James W. Andersen, Project Manager
Millstone Project Directorate
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-423

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cc w/enc: See next page

REQUEST FOR ADDITIONAL INFORMATION

MILLSTONE NUCLEAR POWER STATION, UNIT 3

DOCKET NO. 50-423

1. In the safety evaluation supporting License Amendment No. 87, page 7, the staff stated that:

The licensee now notes, in the "Bases" for TS [Technical Specification] 3.4.6.6.1.d that this change is made "In order to ensure a negative pressure in all areas inside the secondary containment boundary under most meteorological conditions, the negative pressure acceptance criteria at the measured location is 0.4 inches water gauge (WG)." The licensee goes on to state, "It is recognized that there will be an occasional meteorological condition under which slightly positive pressure may exist at some localized portions of the boundary (e.g., the upper elevations on the down wind side of a building). For example, a very low outside temperature combined with a moderate wind speed could cause a slightly positive pressure at the upper elevations of the containment enclosure building on the leeward face. The probability of occurrence of meteorological conditions which could result in such a positive differential pressure condition in the upper levels of the enclosure building has been estimated to be less than 2% of the time.

The staff further stated on page 7:

The licensee has conducted four tests with the new equipment setup to simulate winter (2 tests) and summer (2 tests). The staff reviewed these tests and determined that the subatmospheric pressure in all areas controlled by SLCRS [supplementary leak collection and release system] was reduced to a value below that desired (-0.4-inch WG) within the 120-second period. These tests also indicated that the value attained at the 24-feet, 6-inch level in the auxiliary building was higher (less negative) than or approximately equivalent to that in the other buildings.

In the Sargent & Lundy (S&L) letter dated June 16, 1998, DR-MP3-0514 raises the concern that the single point differential measurement in the auxiliary building is not representative of the containment enclosure differential pressure in all cases. The S&L letter specifically discusses the main steam valve building. Please address S&L's comment (No. 2) under the "Comments on Second and Supplemental Responses" section. Is the single testing location (24-feet, 6-inch elevation in the auxiliary building) still considered adequate and representative of the entire secondary containment as discussed in Final Safety Analysis Report (FSAR) Section 6.2.3.3?

Enclosure

2. FSAR Section 6.2.3.3 states that the surveillance verifies that one train of SLCRS in conjunction with the auxiliary building filtration system will produce a negative pressure of -0.4-inch WG at the auxiliary building's 24-foot, 6-inch elevation relative to the outside atmosphere in the secondary containment. In S&L's letter dated June 16, 1998, DR-MP3-0580 raises issues regarding the ventilation fans running during the surveillance and the effect on the results of the surveillance.
 - a. Please describe the Millstone Unit 3 surveillance testing procedure as it relates to fan operation during the test.
 - b. Are the initial operating conditions for the surveillance test specified or consistently established?
 - c. If the initial operating conditions are not consistent, how is degradation of secondary containment integrity trended?
 - d. If the initial operating conditions are consistently established, describe how degradation of secondary containment integrity is trended.

Millstone Nuclear Power Station
Unit 3

cc:

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Millstone Nuclear Power Station
Unit 3

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