

Nebraska Public Power District

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CNSS886131

June 2, 1988

U.S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, D.C. 20555

Subject: Response to NRC Bulletin No. 85-03, Supplement 1

Cooper Nuclear Station NRC Docket No. 50-298

- References: 1) Letter from R. F. Janecek (BWR Owners' Group) to J. H. Sniezek (NRC), BWROG 8815/OTR1, dated March 28, 1988
 - 2) Letter from J. M. Pilant to R. D. Martin dated November 26, 1986, "Revised Response to IE Bulletin No. 85-03"

Gentlemen:

Supplement 1 to IE Bulletin No. 85-03 directed BWR licensees to include inadvertent mispositioning of safety-related motor-operated valves in certain high pressure systems within the scope of the bulletin. Any valves added as a result of this scope change were to be reported to the Commission, along with the pertinent information discussed in the bulletin. Prior to its issuance, the intent of the supplement was discussed with the NRC staff by the BWR Owners' Group of which the District is a member. Based on these discussions and a review of generic BWR high pressure piping configurations, nine (9) valves were identified by the Owners' Group to be added to the scope of the bulletin (Reference 1).

A review of these additional valves determined that only four (4) were applicable to Cooper Nuclear Station. These four valves (HPCI-MOV-MO17. HPCI-MOV-MO20, RCIC-MOV-MO18, and RCIC-MOV-MO20) were already included in the District's previous response to the bulletin as contained in Reference 2. As a result, the District considers its previous response to encompass the scope of Supplement 1 to the Bulletin and no additional valves will be added to our test program.

Per our original schedule, IE Bulletin 85-03 valve testing will be completed during the 1988 Spring Refueling Outage and the Phase II report will be submitted within 60 days of the completion of testing. No further submittals are expected after the Phase II report, provided no additional guidance is received that would necessitate additional testing.

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Finally, enclosed is a revision to Table 2-1 (Reference 2) that changes the maximum expected operational differential pressure for five valves. The affected valves are indicated by revision bars in the margin and the new differential pressure figures are generally more conservative than before. These revised figures will be used in calculating the necessary valve thrust requirements used to determine torque switch settings.

If you have any questions or require further information, please call.

L. G. Kuhcl

Nuclear Power Group Manager

LGK/gmc:rs(2E) Enclosure

cc: Regional Administrator

NRC Region IV Arlington, Texas

NRC Senior Resident Inspector Cooper Nuclear Station

NPPD-CNS IEB85-03 RESPONSE TABLE 2-1

MOV Data Summary Table (RCIC)

COMPONENT IDENTIFICATION CODE (CIC)	COMPONENT DESCRIPTION	OPERATIONAL REQUIREMENTS	DESIGN DIFFERENTIAL PRESSURE	MAX. EXPECTED OPERATIONAL DIFFERENTIAL PRESSURE	REQUIRED ACTIVE OPERATION	COMMENT
RCIC-MOV-M015	RCIC Steam Inboard Isclation	Open for Core Cooling Supply Close for System Steam Line Isolation	1146 psi	1091 psi	Close	Valve not required to be tested at maximum Δ P
RCIC-MOV-M016	RCIC Steam Outboard Isolation	Open for Core Cooling Supply Close for System Steam Line Isolation	Il46 psi	1091 psi	Close	Valve not required to be tested at maximum Δ P
RCIC-MOV-M018	RCIC Supply from Emergency Conden- sate Storage Tank	Open/Close for Core Cooling Supply	50 psi	14 psi	Close	
RCIC-MOV-MO20	System Pump Discharge Block	Open for Core Cooling Supply	1925 psi	1222 psi	None	Valve not required to be tested at maximum Δ P
RCIC-MOV-MO21	RCIC Injection to Reactor	Open for Core Cooling Supply	1925 psi	1222 psi	Open	
RCIC-MOV-M027	RCIC Pump Minimum Flow Recirc to Torus	Close/Open for Core Cooling Supply	1500 psi	1335 psi 1338 psi	Open Close	
RCIC-MOV-M030	RCIC Test Return to ECST	Close for Core Cooling Supply	1925 psi	1299 pui	None	Valve not required to be tested at maximum Δ P
RCIC-MOV-MO41	RCIC Supply from Torus	Close/Open for Core Cooling Supply	50 psi	88 psi 65 psi	Open Close	
RCIC-MOV-MO131	RCIC Steam Supply to RCIC Turbine	Open for Core Cooling Supply	1146 psi	1091 psi	Open	
RCIC-MOV-M0132	Auxiliary Cooling Supply	Open for Core Cooling Supply	1500 psi	1311 psi	0pen	

NPPD-CNS IEB85-03 RESPONSE TABLE 2-1 MOV Data Summary Table (HPCI)

CGMPONENT IDENTIFICATION CODE (CIC)	COMPONENT DESCRIPTION	OPERATIONAL REQUIREMENTS	DESIGN DIFFERENTIAL PRESSURE	MAX. EXPECTED OPERATIONAL DIFFERENTIAL PRESSURE	REQUIRED ACTIVE OPERATION	COMMENT
HPCI-MOV-MO14	Steam Supply to Turbine	Open for Core Cooling Supply	1146 psi	1091 psi	Open	
HPCI-MOV-M 15	Steam Supply Inboard Isolation	Open for Core Cooling Supply Close for System Steam Line Isolation	1146 psi	1091 psi	Close	Valve not required to be tested at maximum Δ P
HPCI-MOV-M016	Steam Supply Outboard Isolation	Open for Core Cooling Supply Close for System Steam Line Isolation	1146 psi	1091 psi	Close	Valve not required to be tested at maximum Δ P
HPCI-MOV-MO17	Pump Suction from Emergency Conder- sate Storage Tank	Open/Close for Core Cooling Supply	150 psi	14 ps1	Close	
HPCI-MOV-M019	HPCI Injection	Open/Close for Core Cooling Supply	1325 psi	1118 psi	Open	
HPCI-MOV-MO20	Steam Pump Discharge Block	Open for Core Cooling Supply	1925 psi	1118 psi	None	Valve not required to be tested at maximum Δ P
HPCF-MOV-MO21	HPCI Pump Test Bypass to ECST	Close for Core Cooling Supply	1925 psi	1284 psi	None	Valve not required to be tested at maximum Δ P
HPCI-MOV-MO25	HPCI-Pump Minimum Flow Bypass Line Isolation	Close/Open for Core Cooling Supply	1500 psi	1326 psi 1340 psi	Open Close	
HPCI-MOV-M058	HPCI Pump Suction from Suppression Pool	Close/Open for Core Cooling Supply	150 psi	98 psi 64 psi	Open Close	