NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Palo Verde Unit 1	0 5 0 0 0 5 2 8	1 OF 0 2

TITLE (4)
Steam Generator Blowdown System Snubbers Not Functionally Tested 12 Months Prior to Installation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 8	1 9	8 5	8 5	0 6 9	0 1	0 3	2 7	8 6		0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
3		20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 0 0		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
William F. Quinn, Manager - Nuclear Licensing (Extension 4087)	6 0 2 9 4 3 - 7 2 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

This is a supplement to LER 85-069-00.

On August 19, 1985, a review of snubber operability revealed that fifteen snubbers in the Steam Generator Blowdown System (WI) had not been functionally tested within 12 months prior to installation as required by Technical Specification (T.S.) 4.7.9.(h). Administrative controls were not in place to prevent this from occurring.

This review of snubber operability was continued after previously finding five untested snubbers in the letdown portion of the Chemical and Volume Control System (CVCS)(CB) as reported in LER 85-060-00.

All the snubbers were replaced with functionally tested snubbers as required by T.S. 4.7.9.(h).

To prevent recurrence, plant procedures have been reviewed and revised to ensure inclusion of controls which limit installation of Technical Specification Snubbers to those which have been functionally tested in accordance with T.S. 4.7.9.(h).

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Palo Verde Unit 1	0 5 0 0 0 5 2 8	8 5	— 0 6 9	— 0 1	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This is a supplement to LER 85-069-00.

At 1725 on August 19, 1985, while Palo Verde Unit 1 was in Mode 3 at 515 F and 2250 psia, an engineering review of snubber operability revealed that fifteen snubbers in the Steam Generator Blowdown System (WI) had not been functionally tested within 12 months prior to installation as required by Technical Specification (T.S.) 4.7.9.(h). Administrative Controls were not in place to prevent this from occurring.

This review of snubber operability was continued after previously finding five untested snubbers in the letdown portion of the Chemical and Volume Control System (CVCS)(CB) as reported in LER 85-060-00.

The snubbers were all replaced with snubbers that had been functionally tested within 12 months prior to installation. Four of the snubbers were replaced on August 21, 1985, and eleven were replaced on August 23 and 24, 1985.

Eleven of the snubbers that were removed were functionally tested and were determined to be within specification. These eleven snubbers, though not meeting surveillance requirements, were functional and would have had no adverse safety impact on the plant. The remaining four snubbers were sent offsite and tested. Three of the snubbers passed the functional test and, therefore, would have had no adverse safety impact on the plant. One snubber exceeded the drag force limit by 0.5 lb. The cause of the excessive drag force was determined to be a slightly bent screw shaft. The snubber was repaired and sent back. An engineering evaluation determined that there were no detrimental effects on the system and no adverse safety impact on the plant.

To prevent recurrence, plant procedures have been reviewed and revised to ensure inclusion of controls which limit installation of Technical Specification Snubbers to those which have been functionally tested in accordance with T.S. 4.7.9.(h).

Arizona Nuclear Power Project
P.O. BOX 52034 • PHOENIX, ARIZONA 85072-2034

March 27, 1986
ANPP-35762-EEVB/PGN/98.05

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Palo Verde Nuclear Generating Station (PVNGS)
Unit 1
Docket No. STN 50-528 (License NPF-41)
Licensee Event Report - 85-069-01
File: 86-020-404

Dear Sirs:

Attached please find Supplement Number 01 to Licensee Event Report (LER) No. 85-069-00 prepared and submitted pursuant to 10 CFR 50.73. In accordance with 10 CFR 50.73(d), we are herewith forwarding a copy of this report to the Regional Administrator of the Region V Office.

If you have any questions, please contact me.

Very truly yours,

E. E. Van Brunt

E. E. Van Brunt, Jr.
Executive Vice President
Project Director

EEVB/PGN/rw
Attachment

cc: J. B. Martin (all w/a)
R. P. Zimmerman
A. L. Hon
E. A. Licitra
A. C. Gehr
INPO Records Center

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