



March 17, 1986

Northern States Power Company

414 Nicollet Mall Minneapolis, Minnesota 55401 Telephone (612) 330-5500

Director Office of Nuclear Reactor Regulation U S Nuclear Regulatory Commission Washington, DC 20555

> PRATRIE ISLAND NUCLEAR GENERATING PLANT Docket Nos. 50-282 License Nos. DPR-42 50-306 DPR-60

License Amendment Request dated March 17, 1986 LCO Action Statements and Miscellaneous Changes

Attached are three originals and thirty-seven conformed copies of a request to change the Technical Specifications, Appendix A, of Operating Licenses DPR-42 and DPR-60. In accordance with 10 CFR Part 170, a check in the amount of \$150.00 is enclosed.

This License Amendment Request contains a major revision to the Prairie Island Technical Specifications. Exhibit A describes the proposed changes, reasons for change and the significant hazards evaluation. Exhibit B contains the revised Technical Specification pages.

In response to a recommendation from the Prairie Island NRC Resident Inspector, a Technical Specification revision was initiated to incorporate a generic action statement. This generic action statement would be followed when required actions are not specified, or cannot be satisfied. In conjunction with this revision, a thorough review of the Prairie island Technical Specifications was made to ensure the compatibility of the generic action statement with the existing Technical Specifications. During the course of this review it became apparent that additional improvements to action statements should be made.

Since the action statement improvements affected a large number of Technical Specification sections, it was an ideal time to also incorporate changes which would make the Technical Specifications easier for Plant and NRC personnel to use and which would reduce the potential for misinterpretation or human error. These changes support the ongoing human error reduction program at Prairie Island. The resulting changes include the reorganization and standardization of some sections, the removal of ambiguities, and changes designed to ensure that a consistent format is utilized throughout the Technical Specifications. However, the original format of the Prairie Island Technical Specifications was retained in order to minimize the amount of retraining for operations personnel as well as others who use the Technical Specifications.

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The changes noted above and other miscellaneous Technical Specification changes included in this License Amendment Request, are listed on the first page of Exhibit A.

Some of the format changes made to section 3.0 are also applicable to section 4.0. However, all pages of Section 4.0 are not included in this change, but will be revised in a future change.

Please contact us if you have any question concerning this License Amendment Request. Due to the magnitude of this change and its affect on plant procedures and training, please make this change effective 90 days following its approval. This will allow time to complete plant procedure revision and operations personnel training.

David Musolf

D. OM

Manager - Nuclear Suppord Services

DMM/EFE/tp

c: Regional Administrator-III, NRC NRR Project Manager, NRC Resident Inspector, NRC MPCA

Attn: F W Ferman

G Charnoff

Attachments: Affidavit of David Musolf

Exhibit A. Evaluation of Proposed Changes

Exhibit B, Proposed Changes

Exhibit C, Safety Evaluation of 7.0% vs 6.9% Negative

Rate Trip Setpoint

Check (Attached to Director of NRR's copy)