ALABAMA POWER COMPANY
JOSEPH M. FARLEY NUCLEAR PLANT
UNIT 2 - ANNUAL REPORT
REQUIRED BY 10CFR50.59 (1985)

Section 50.59 of Part 50, Licensing of Production and Utilization Facilities, of the regulations of the United States Nuclear Regulatory Commission, states that the holder of a license authorizing operation of a production or utilization facility may (1) make changes in the facility as described in the safety analysis report, (2) make changes in the procedures as described in the safety analysis report, and (3) conduct tests or experiments not described in the safety analysis report, without prior commission approval, unless the proposed change, test or experiment involves a change in the technical specifications incorporated in the license or an unreviewed safety question (as defined in 10CFR50.59).

The licensee is required to maintain records of such changes, tests or experiments, and these records are required to include written safety evaluations which provide the basis for the determination that the change, test or experiment does not involve an unreviewed safety question.

Brief descriptions and a summary of the safety evaluations of the changes, tests or experiments as described above which were completed in 1985 for the Joseph M. Farley Nuclear Plant Unit 2, are provided in the following pages.

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Subject:	Unit 2 Cycle 4 Reload Safety Evaluation
Description:	The Joseph M. Farley Unit 2 Cycle 4 fuel reload consisted of 21 Region 4, 64 Region 5, and 72 fresh Region 6 fuel assemblies. The design was an 18-month cycle and included 432 fresh burnable poison rodlets comprising 40 burnable poison clusters containing groups of 8 and 12 burnable poison rodlets.
Safety Evaluation:	A Reload Safety Evaluation was conducted by Westinghouse. The reload design was found to be in compliance with current Technical Specifications and to involve no unreviewed safety question per 10CFR50.59.
PORC Review:	PORC Meeting 1319, 12/4/84
Subject:	FNP-2-AOP-2.0, Revision 2, TCN 2A
Description:	The Final Safety Analysis Report indicates that the steam generator blowdown isolation valves close automatically on a high radiation signal from radiation monitor R-19 in addition to signals from R-23A and R-23B. This is contrary to design drawings and actual tests which indicate that these valves do not close on a signal from R-19. This procedure was changed to reflect the actual design of the plant.
Safety Evaluation:	This change was made to reflect the existing design of the plant. The existing design was determined to be the proper design.
PORC Review:	PORC Meeting 1450, 11/7/85
Subject:	FNP-0-AP-2, Revision 9
Description:	Revised procedure to show new membership of the Plant Operations Review Committee (PORC) due to plant management changes and made minor changes in the review requirements of the PORC.
Safety Evaluation:	These changes increase the effectiveness of the PORC and plant management.
PORC Review:	PORC Meeting 1370, 3/28/85

Subject:	FNP-O-AP-3, Revision 4 "Plant Organization"
Description:	Revised procedure to show the new plant management and changed the educational requirements of the Systems Performance Supervisor.
Safety Evaluation:	These changes increase the effectiveness of the plant management.
PORC Review:	PORC Meeting 1370, 3/28/85
Subject:	FNP-O-AP-45, Revision 5 "Farley Nuclear Plant Training Plan"
Description:	Revised procedure to not allow exempting operators from annual retraining based on previous year's examination performance.
Safety Evaluation:	This change increases the effectiveness of the training program.
PORC Review:	PORC Meeting 1371, 3/29/85
Subject:	FNP-2-ETP-1009
Description:	This procedure was performed to determine the affect of the blockage of certain residual heat removal (RHR) system pump room penetrations upon the ability of the HVAC systems to maintain negative pressure in the room.
Safety Evaluation:	Since this test involved the potential isolation of the RHR pump room from its post-LOCA filtration system, analyses were performed which demonstrated that the performance of the test could not result in a condition in which the plant was outside of the bounds of the accidents analyzed in the Final Safety Analysis Report. This procedure also allowed the use of a smoke generator to perform leak checks and air flow evaluations. The smoke is generated by a chemical reaction which does not involve combustion. It is not corrosive and it is not harmful to personnel or filtration systems.
PORC Review:	PORC Meeting 1330, 1/3/85 PORC Meeting 1353, 2/14/85

Subject: FNP-2-ETP-1010, Revision 0
"Testing of Rod Control System"

Description: This procedure allows withdrawal of rod banks out of their normal sequence--one bank at a time--for the purpose of isolating and detecting maintenance problems. It also allows withdrawal of individual rods within a bank.

Safety Evaluation: The procedure is not considered to increase the probability or worsen the consequences of any accident for the following reasons.

1) Keff will be maintained at < 0.99 during the procedure.
2) A $^{1}/M$ Plot will be performed during the test.
3) Proper shutdown margin will be verified to be maintained prior to pulling any control rod bank or individual rod.

PORC Review: PORC Meeting 1416, 8/4/85

Subject: FNP-2-ETP-4147, Revision 0

Description: Procedure for installation of nozzle dams in the steam generators. In the past during refueling, steam generator channel head work on Unit 2 was scheduled before and after refueling with the refueling cavity drained to mid-loop.

With nozzle dams installed, channel head work can be scheduled in parallel with fuel movement while the cavity is fully flooded.

Safety Evaluation: The safety evaluation for this procedure was based on the following Westinghouse safety evaluations:

SG Primary Nozzle Dam User's Manual

Installation and Removal of Temporary Nozzle Covers

Pre-Installation Hydro Test of SG Primary Nozzle Dams for Alabama Power Company's (APCo) Farley Units 1 and 2

Vent Hole Plugging SG Hold Down Rings

SG Primary Nozzle Dam Installation/Removal for APCo Units 1 and 2

Nozzle Dam System (Electrical) User's Manual for APCo Units 1 and 2

Based on these evaluations, it was determined that the use of nozzle dams for Unit 2 does not constitute an unreviewed safety question.

PORC Review: PORC Meeting 1334, 1/7/85

Subject:	FNP-0-FHP-1.0, Revision 10
Description:	This procedure provides the method used to remove the reactor vessel head during refueling. This method involves removing the reactor vessel head and placing it on the storage stand prior to flooding the refueling cavity. This is different from the procedure described in the Final Safety Analysis Report in which the head is raised while simultaneously flooding the cavity.
Safety Evaluation:	This method reduces the radiation exposure involved by shortening the duration of the job. The extremely difficult coordination effort of raising the head and cavity level simultaneously is eliminated. The time the head is suspended from the polar crane is reduced.
PORC Review:	PORC Meeting 1400, 6/18/85

Subject: FNP-O-FHP-3.1, Revision 9, TCN OB

Description: The final Safety Analysis Report states that new fuel assemblies are moved in the new fuel area by use of the new fuel assembly handling fixture suspended from the new fuel hoist. This procedure change allowed for the use of slings to lift a dropped fuel assembly from the horizontal position since the handling fixture could not be used.

Safety Evaluation: The handling of a single damaged unburned assembly is safe from a criticality standpoint. The undamaged assembly is the most reactive geometry, i.e., the assembly will not become significantly more reactive if it is further deformed when it is lifted and placed in the shipping container.

PORC Review: PORC Meeting 1458, 11/26/85

Subject: FNP-2-SOP-50.0, Revision 11

Description: During a review of this procedure, it was found that the Waste Evaporator Feed Pump and Floor Drain Tank pump recirc valves were listed as "locked throttled" which was contrary to system drawings and the Final Safety Analysis Report (FSAR). Upon investigation it was found that "locked throttled" was the desired condition and that the system drawings and FSAR should be changed. Further, this revision changed the normal required positions for the following valves from "locked open" to "locked closed":

Waste Evaporator Concentrate Tank pump recirc valve

Chemical Drain Tank pump recirc valve

Laundry and Hot Shower Tank pump recirc valve

Safety Evaluation: The purpose of the recirc path is to provide minimum flow protection for the pumps and recirc mixing. According to startup test procedures, the recirc valves for all waste processing system pumps should have been throttled to obtain a nominal recirc flow sufficient for pump protection and recirc mixing.

PORC Review: PORC Meeting 1350, 2/8/85

Subject:	FNP-2-SOP-50.0, Revision 11, TCN 11C, TCN 11D
Description:	Changed the normal required positions for the Reactor Coolant Drain Tank (RCDT) pump discharge manual isolation valves and the RCDT pump recirc orifice outlet isolation valve from "locked open" to "locked throttled". Having these valves in the locked open position has resulted in the RCDT pumps exceeding design flow. Also, having the recirc valve wide open was causing insufficient head for transfering tank contents through the discharge path.
Safety Evaluation:	The valves have been throttled to conform with pump design and pre-operational testing results.
PORC Review:	PORC Meeting 1389, 5/14/85 PORC Meeting 1433, 9/17/85
Subject:	FNP-2-UOP-2.2, Revision 11
Description:	Changed this unit operating procedure to require that the residual heat removal (RHR) system suction isolation valves be open and have power to their operators removed when the reactor coolant system (RCS) cold leg temperature is less than 180 degrees.
Safety Evaluation:	This is to prevent a spurious isolation signal from closing these valves during RHR system operation resulting in isolation of the RCS overpressurization protection relief valves which are located in the RHR system. The procedure included the necessary steps for restoration of power to the RHR suction isolation valves.
PORC Review:	PORC Meeting 1332, 1/4/85
Subject:	MD 85-1253
Description:	Installed "0.01" Beckman conductivity cells for measuring steam generator blowdown cation conductivity in existing sample lines.
Safety Evaluation:	This change does not adversely affect sample system operation. The panel in which these cells are located is isolated during a radiological accident.
PORC Review:	PORC Meeting 1456, 11/21/85

* Subject:	PCR/PCN 81-0-1037-1A-132 (SM)
Description:	Installed pre-action sprinkler system 1A-132 in auxiliary building room 107 to facilitate use of this room as an occasional combustible materials storage area. Also the room boundary was upgraded to a 3-hour fire rating.
Safety Evaluation:	This design was evaluated for its effect on other systems and was found not to interfere with or degrade other installed systems.
PORC Review:	PORC Meeting 1443, 10/10/85
* Subject:	PCR/PCN 81-0-1037-1A-134 (SM)
Description:	Installed pre-action sprinkler system 1A-134 to cover auxiliary building rooms 601, 602, 603, 2601, 2602, and 2603 to facilitate use of these rooms for occasional combustible material storage.
Safety Evaluation:	This design has been evaluated for its effect on other systems and has been found not to interfere with or degrade other installed systems.
PORC Review:	PORC Meeting 1443, 10/10/85
* Subject:	PCR/PCN 81-0-1037-603 (B81-0-1037-603)
Description:	Upgraded rooms 601, 602, 603, 2601, 2602, and 2603 to be a common 3 hour rated fire area for use as combustible materials storage area.
Safety Evaluation:	This design has been evaluated for its effect on other systems and has been found not to interfere with or degrade other installed systems.
PORC Review:	PORC Meeting 1415, 8/1/85
* Subject:	PCR/PCN 81-1167 (S81-0-1167)
Description:	Piping and associated valves were added in order to connect the Unit 2 service water standpipe inlet to the fire pump suction header.
Safety Evaluation:	This addition provides a supply of water to the fire pumps in case the fire protection storage tanks are depleted for any reason and has no adverse effect on service water system operation.
PORC Review:	PORC Meeting 1297, 10/9/84

Subject:	PCR/PCN 81-2063 (S81-2063)
Description:	Added Atlas-Copco rotary-screw air compressor to the compressed air system.
Safety Evaluation:	The addition of this new air compressor improves system operation without degrading system safety.
PORC Review:	PORC Meeting 1146, 8/25/83 PORC Meeting 1162, 9/29/83 PORC Meeting 1219, 3/01/84 PORC Meeting 1272, 8/17/84 PORC Meeting 1337, 1/10/85
Subject:	PCR/PCN 81-2072 (S81-2-2072)
Description:	Added a standpipe and surge tank to the service water system in order to maintain backpressure without the use of throttling valves.
Safety Evaluation:	The standpipe and surge tank are assumed to fail following a seismic event in which case service water bypasses them and is returned to the pond. Adequate system backpressure is then maintained by resistance in the piping.
PORC Review:	PORC Meeting 1287, 9/18/84 PORC Meeting 1323, 12/18/84
* Subject:	PCR/PCN 82-2216 (S82-2-2216)
Description:	Installed two wet pipe sprinkler systems (2T-5 and 2T-7) in the Unit 2 side of the turbine building.
Safety Evaluation:	This modification minimizes the impact of a fire fueled by a major spill of turbine lube oil, thus enhancing plant safety. The design has been evaluated for its effect on other systems and has been found not to interfere with or degrade other installed systems.
PORC Review:	PORC Meeting 1310, 11/8/84 PORC Meeting 1443, 10/10/85

Subject:	PCR/PCN 82-2226 (B82-2226)
Description:	Modified annunciation circuit for loss of voltage tripping buses 2F, 2G, 2H, 2J, and degraded grid voltage tripping buses 2F, 2G to provide immediate targeted information to aid in troubleshooting a tripped bus.
Safety Evaluation:	This modification in no way adversly affects the safe operation of the plant.
PORC Review:	PORC Meeting 1011, 7/30/82 PORC Meeting 1292, 10/2/84 PORC Meeting 1294, 10/4/84

Subject:	PCR/PCN 82-2259.1 (S82-2-2259.1)
Description:	Replaced the 3A, 3B, 4A, 4B, 6A, and 6B feedwater heaters.
Safety Evaluation:	Copper in the original feedwater heaters is suspected to be a major contributor to steam generator tube denting. The new heaters utilize stainless steel tubes. The new heaters are designed to meet or exceed the old heaters' performance with the operation being similar. Replacement of the feedwater heaters improves the system's performance without degrading system safety. These heater replacements are expected to assist in prolonging steam generator longevity.
PORC Review:	PORC Meeting 1287, 9/18/84

Subject:	PCR/PCN 83-2351 (B83-2351)
Description:	Upgraded the main control room work stations.
Safety Evaluation:	Provides operations personnel with better access to procedures, computer console and greater visibility of the main control board.
PORC Review:	PORC Meeting 1143, 8/16/83

Subject:	PCR/PCN 83-2407 (S83-2-2407)
Description:	Replaced the existing carbon steel service water supply and drain piping and valves for the diesel generator 2B air start system aftercoolers with stainless steel piping and valves.
Safety Evaluation:	This replacement will allow proper cooling water flow to the aftercoolers. This will reduce maintenance and operational problems associated with corrosion of existing carbon steel piping. This modification will not reduce the integrity of the system and does not constitute an unreviewed safety question.
PORC Review:	PORC Meeting 1402, 6/25/85 PORC Meeting 1418, 8/8/85
Subject:	PCR/PCN 83-2417 (S83-2-2417)
Description:	Replace the gland seal steam condenser (GSSC) tubes with stainless steel tubes and added a redundant vapor exhauster on the GSSC.
Safety Evaluation:	This modification does not affect plant safety and increases reliability.
PORC Review:	PORC Meeting 1275, 8/23/84
Subject:	PCR/PCN 83-2-2418 (B83-2-2418)
Description:	Installed a flanged spool piece in each reactor coolant system (RCS) loop drain line downstream of the last drain valve. This will allow for the spool piece to be removed and blank flanges to be installed in the drain lines at any time in case of seat leakage.
Safety Evaluation:	The addition of spool pieces in the RCS drain lines does not compromise the integrity of the RCS pressure boundary since the Safety Class 1 double isolation valves in the drain lines remain intact.
PORC Review:	PORC Meeting 1266, 7/20/84 PORC Meeting 1306, 10/31/84

Subject: PCR/PCN 83-2419 (B83-2-2419)

Description: Replace the existing letdown orifice relief valve Q2E21V255

Safety Evaluation: The letdown orifice relief valve has repeatedly opened and failed to reseat during various plant evolutions involving the letdown orifices. The failure of the valve to reseat causes unnecessary plant shutdowns and delays in startup.

PORC Review: PORC Meeting 1266, 7/20/84

Subject: PCR/PCN 83-2442 (B83-2-2442)

Description: Installed a Gamma-Metrics neutron monitor instrument with display in the hot shutdown panel room.

Safety Evaluation: This change complies with the requirements of 10CFR50 Appendix R and Regulatory Guide 1.97.

PORC Review: PORC Meeting 1269, 8/7/84

Subject: PCR/PCN 83-2-2449 (B83-2-2449)

Description: Provided control capability from the hot shutdown panel for the following components:

Main Steam Isolation Valves	SV3369A, B, and C
Component Cooling Water Heat Exchanger Discharge Valve	MOV-3047
Refueling Water Storage Tank to Charging Pump Suction Valves	LCV-115B, and D
Pressurizer Power Operated Relief Valves (PORVs)	PCV-444B, and 445A
Pressurizer PORV Block Valves	MOV-8000A, and B
Reactor Vessel Head Vent Valves	SV-2213A, B, 2214A, B
Seal Water Injection Valve	HCV-186

Also, reactor coolant system Tcold and condensate storage tank level indication was provided on the hot shutdown panel.

Safety Evaluation: These modifications comply with 10CFR50 Appendix R and increase the safe operation of the plant.

PORC Review: PORC Meeting 1292, 10/2/84

Subject:	PCR/PCN 83-2-2467 (B83-2-2467)
PORC Review:	PORC Meeting 1262, 7/10/84
Description:	Environmentally qualified the containment air cooler intake temperature sensors.
Safety Evaluation:	This change was evaluated for its effect on other systems and it was determined that it does not degrade other installed systems.

Subject:	PCR/PCN 83-2519 (B84-2-2519)
Description:	Added temperature monitoring system to detect check valve leakage in the auxiliary feedwater system.
Safety Evaluation:	This change addresses the issue of auxiliary feedwater pump steam binding due to leakage of system check valves. This change has been evaluated for its effect on other installed systems.
PORC Review:	PORC Meeting 1269, 8/7/74

Subject:	PCR/PCN 84-2565 (S84-2-2565)
Description:	Changed the Unit 2 river water pumps from water to grease lubrication.
Safety Evaluation:	This modification increases the reliability of the river water pumps which had been experiencing problems due to the use of river water, which contains suspended abrasive particulates, for lubrication of the pump bearings.
PORC Review:	PORC Meeting 1402, 6/25/85

Subject:	PCR/PCN 84-2652 (B84-2-2652)
Description:	Modified the Main Steam Isolation Valve (MSIV) test cylinder and associated hardware for MSIV Q2N11V002A to correct problems being encountered with this system.
Safety Evaluation:	This modification does not affect valve weight, center of gravity, seismic calculations, or operability of the valve.
PORC Review:	PORC Meeting 1320, 12/7/84

Subject:	PCR/PCN 84-2-2656 (B84-2-2656)
Description:	Replaced existing flow sensing instrumentation in the battery room exhaust fan ducts.
Safety Evaluation:	The previously existing instrumentation was in constant alarm. The ventilation system was determined to have no design deficiency. The purpose of the flow sensing instrumentation is to ensure fan operation but the actual pressure drop was too low to be sensed by the previously existing instrumentation.
PORC Review:	PORC Meeting 1356, 2/21/85
Subject:	PCR/PCN 84-2678 (S84-0-2678)
Description:	Assigned TPNS numbers to eight thermostats which control the operation of the control room ventilation duct heating systems covering both units.
Safety Evaluation:	This change assigns TPNS numbers to existing thermostats and does not degrade the safe shutdown capabilities of the plant.
PORC Review:	PORC Meeting 1387, 5/7/85
Subject:	PCR/PCN 84-2721 (B84-0-2721)
Description:	Modified the existing HVAC system for the computer room to adequately cool the room upon installation of Safety Parameter Display System (SPDS) equipment.
Safety Evaluation:	This design supports the implementation of the SPDS which is required by NUREG-0737, Supplement 1.
PORC Review:	PORC Meeting 1311, 11/13/84 PORC Meeting 1387, 5/7/85

Subject:	PCR/PCN 84-2724 (B84-2-2724)
Description:	Installed a normally locked open valve in the normal charging flowpath at penetration No. 24 to facilitate the performance of Local Leak Rate Testing (LLRT) of this penetration.
Safety Evaluation:	This new valve replaces several downstream valves as a boundary valve for the LLRT of the containment isolation valves for penetration No. 24. This enhances the ability to perform a meaningful LLRT. Since the valve will be locked open at all times, except during testing, there are no adverse effects on plant operation or safety.
PORC Review:	PORC Meeting 1323, 12/18/84 PORC Meeting 1325, 12/21/84
Subject:	PCR/PCN 84-2778 (S82-2-2778)
Description:	Installed differential pressure gauges across each charging pump oil cooler to supplement existing pressure indicators in detecting developing oil cooler service water side plugging problems.
Safety Evaluation:	The differential pressure indicators will inform plant personnel of developing cooler problems. This change will not degrade the safe shutdown capabilities of the plant.
PORC Review:	PORC Meeting 1306, 10/31/84
Subject:	PCR/PCN 84-2824 (S84-0-2824)
Description:	Installed a vertical scale indicator to give local level indication for the demineralized water storage tank.
Safety Evaluation:	A local level indicator was installed in the tank level transmitter circuit of the demineralized water storage tank due to the existing mechanical level indicator being broken and beyond repair. Since the demineralized water system is not required for any safety-related functions, this change will not degrade the safe shutdown capabilities of the plant.
PORC Review:	PORC Meeting 1387, 5/7/85 PORC Meeting 1452, 11/12/85

Subject:	PCR/PCN 84-2900 (B84-2-2900)
Description:	Modified the Main Steam Isolation Valve (MSIV) Q2N11V002A stuffing box in order to minimize the force on the MSIV shaft that could prevent valve closure and relocated the MSIV position switches to reduce the number of man-hours required to calibrate the position switches.
Safety Evaluation:	This modification does not affect valve weight, center of gravity, seismic calculations, or the operability of the valve.
PORC Review:	PORC Meeting 1320, 12/7/84
Subject:	PCR/PCN 84-2-2952 (B84-2-2952)
Description:	Revised drawing D-206060 to show existing fire doors into cable chases 2249 and 2250.
Safety Evaluation:	This is a drawing change only to accurately reflect the as-built condition.
PORC Review:	PORC Meeting 1387, 5/7/85
Subject:	PCR/PCN 84-2-3007 (B84-2-3007)
Description:	Replaced the existing main feedwater bypass control valve's trim circuitry with new equal percentage valve trim circuitry in accordance with Westinghouse recommendations for resolving oscillation problems with the feedwater control system.
Safety Evaluation:	This modification did not result in a "change to the plant as described in the Final Safety Analysis Report (FSAR)"; however, testing of the system during implementation involved a "test or experiment not described in the FSAR." This testing involved the addition of temporary test equipment. This was evaluated and it was determined that, since no safety-related changes were being made, i.e., the existing protection function of these valves is maintained, this test is acceptable and has no safety impact.
PORC Review:	PORC Meeting 1334, 1/7/85

Subject:	PCR/PCN 84-3020 (B84-0-3020)
Description:	Removed the interlock mechanism on Motor Control Center (MCC) "F" cubicle door which tripped the MCC if the door was opened.
Safety Evaluation:	This design change was accomplished within the aplicable electrical codes and standards. This change prevents the loss of safety-related loads if the cubicle door is inadvertently opened.
PORC Review:	PORC Meeting 1408, 7/19/85
Subject:	PCR/PCN 84-3021 (B84-0-3021)
Description:	Removed the interlock mechanism on Motor Control Center (MCC) "G" cubicle door which tripped the MCC if the door was opened.
Safety Evaluation:	This design change was accomplished within the applicable electrical codes and standards. This change prevents the loss of safety-related loads if the cubicle door is inadvertently opened.
PORC Review:	PORC Meeting 1408, 7/19/85 PORC Meeting 1434, 9/19/85
Subject:	PCR/PCN 84-3028 (B84-2-3028)
Description:	Updated drawing D-205009 sheet 3 to show pressure indicator isolation valves PVGV-1, PVGV-2 and an argon isolation valve downstream of the argon cylinder pressure control valve and upstream of the drierite filter. Also assigned TPNS numbers for these valves.
Safety Evaluation:	The indicated changes to drawing D-205009 sheet 3 are needed to correspond to the actual system design.
PORC Review:	PORC Meeting 1387, 5/7/85

Subject:	PCR/PCN 84-3038 (B84-2-3038)
Description:	Locksets on several Unit 2 doors were changed.
Safety Evaluation:	This change involved no unreviewed safety question.
PORC Review:	PORC Meeting 1387, 5/7/85
Subject:	PCR/PCN 85-3092 (S85-2-3092)
Description:	Changed drawing D-175042 sheets 1, 2, 3, and 4 to show processing waste water via the Disposable Demineralizer System as the primary method of waste treatment.
Safety Evaluation:	This change is made to update drawings for normal method of operation. It does not impact the safe operation of the plant. This change does not constitute an unreviewed safety question.
PORC Review:	PORC Meeting 1400, 6/18/85
Subject:	PCR/PCN 85-3155 (B85-2-3155)
Description:	Since automatic control is not used on service water to component cooling water heat exchanger valves Q2P16V3009A, B, and C, the controller cards for each loop, except the final driver cards, were removed.
Safety Evaluation:	This change did not involve an unreviewed safety question.
PORC Review:	PORC Meeting 1408, 7/19/85
Subject:	PCR/PCN 85-3162 (B85-2-3162)
Description:	Realigned the containment venting system manual isolation valve from "normally closed" to "normally open".
Safety Evaluation:	This change does not impact the design of the system with respect to any of the system design bases including containment isolation. This change eliminates the need to make a "reentry" during postulated accident conditions to manually open this valve to allow containment venting for hydrogen concentration control.
PORC Review:	PORC Meeting 1405, 7/9/85

Subject:	PCR/PCN 85-3176 (S85-2-3176)
Description:	Revised Drawing D-205042 sheets 2, 3, and 4 to change the required position from locked open to locked throttled for recirc valves Q2G21V028, V062, V071, V128, V099A, V099B.
Safety Evaluation:	The purpose of the recirc path is to provide minimum flow protection for the pump and to allow for recirculation mixing of the tank contents. It is not necessary to have the recirc valve fully open to provide adequate pump protection, and with the valve throttled, there is still sufficient recirc flow for tank mixing. This modification does not impact the safe operation of the plant. This modification does not constitute an unreviewed safety question.
PORC Review:	PORC Meeting 1400, 6/18/85
Subject:	PCR/PCN 85-3252 (S85-2-3252)
Description:	Corrected inaccuracies in drawing D-200189, Fire Protection and Detection Systems.
Safety Evaluation:	This change was evaluated for its effect on other systems and was found not to interfere with or degrade other installed systems.
PORC Review:	PORC Meeting 1457, 11/26/85
Subject:	Steam Generator Primary Side Tube Removal
Description:	A steam generator tube was removed (partially) for testing using procedures for which the following Westinghouse safety evaluations were applicable: Steam Generator Primary Side Tube Removal TIG II Tube Relaxation Field Procedure "Hands-On" Machining of Steam Generator Tubes Weld Repair of Steam Generator Tube/Plugs Steam Generator Barehole/Tube Plugging by Gas Tungsten Arc Welding Mechanical Weld Dam Plug with Sleeve
Safety Evaluation:	These procedures were reviewed and determined not to involve an unreviewed safety question.
PORC Review:	PORC Meeting 1356, 2/21/85 PORC Meeting 1358 2/23/85

Subject:	R34C44 Partial Tube Removal Condition
Description:	During the last refueling outage, actions were taken to remove steam generator tube R34C44 on the hot-leg side of steam generator A for the purpose of investigating the significance of eddy current examinations. However, the tube could be only partially removed.
Safety Evaluation:	The remaining portion of the tube and the adjacent tubes were plugged. Westinghouse performed an analysis of this partial tube removal condition that demonstrated the tube bundle integrity for operation until the next scheduled refueling outage at which time an assessment of the condition of the adjacent tubes will be made.
PORC Review:	PORC Meeting 1359, 2/25/85

Subject:	GO-NG-1, Rev. 6 Procedure Development
Description:	This procedure provides guidance for writing a general office procedure.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG, Rev. 4 Organization and Responsibilities
Description:	This procedure decribes the general organization and responsibilities of the offsite support organization.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG, Rev. 2 Master Bid List Input
Description:	This procedure provides guidance for adding vendors to the Master Bid List.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-4, Rev. 3 Specification Review
Description:	This procedure provides guidance for reviewing equipment specifications.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A

Subject:	GO-NG-5, Rev. 2 Environmental Monitoring
Description:	This procedure provides guidance for conducting an environmental monitoring program.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-9, Rev. 3 Plant Services Approved Suppliers List
Description:	This procedure provides guidance for developing and maintaining a Plant Services Approved Suppliers List.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-12, Rev. 9 Procurement Document Control
Description:	This procedure provides guidance for maintaining control of equipment procurement documentation.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-22, Rev. 3 Maintenance Support Conduct of Operation
Description:	This procedure describes the responsibilities of the maintenance support organization.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A

Subject:	GO-NG-33, Rev. 5 Operational Procurement Interfaces
Description:	This procedure describes the interfaces between Nuclear Support, Safety Audit and Engineering Review and the Purchasing Department required for procurement related activities.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-44, Rev. 2 Inservice Inspection
Description:	This procedure provides guidance for conducting an inservice inspection program.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-45, Rev. 0 Technical Support Conduct of Operations
Description:	This procedure describes the responsibilities of the technical support organization.
Safety Evaluation:	This procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A

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R. P. McDonald
Senior Vice President
Flintridge Building

Alabama Power

March 31, 1986

Docket No. 50-364

Dr. J. N. Grace, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street, N.W.
Atlanta, Georgia 30303

Dear Dr. Grace:

Attached for your review is the annual report required by 10CFR50.59 for 1985. This report summarizes changes to the plant performed in accordance with the provisions of 10CFR50.59 for Joseph M. Farley Nuclear Plant Unit 2. Also, in accordance with 10CFR50.59(b), 39 additional copies are provided for your use.

If you have any questions, please advise.

Respectfully submitted,

R. P. McDonald

RPM/JLO:ddb-D6

Attachment

cc: Mr. L. B. Long
Mr. E. A. Reeves
Mr. W. H. Bradford
Director, Office of Inspection and Enforcement, Washington, D.C. 20555
Document Control Desk
Washington, D.C. 20555

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