At the beginning of the report period, Oyster Creek was operating at approximately 650 MWe. Drywell unidentified leak rate, including torus inleakage, began increasing at the end of November and stabilized at 3.5 gpm.

On December 15, a reactor scram occurred due to high neutron flux following a pressure spike caused by closure of the turbine control valves. Closure of the valves resulted from a loose connection in the position indication circuitry associated with turbine controls.

Following plant shutdown, the drywell was purged and an entry conducted to determine the cause of elevated drywell leak rate and effect necessary repairs. The major contributor was determined to be leakage from the bonnet of manual feedwater isolation valve V-2-36. Repairs were performed using sealant injection.

Following completion of required repairs and surveillances, reactor startup commenced on December 16. Primary containment was inerted and the generator placed on-line on December 17. Plant load was increased to 560 MWe by the end of the day. Power was subsequently increased to approximately 650 MWe and maintained for the balance of the report period. Drywell unidentified leak rate, including torus inleakage, remained stable at about 1.0 gpm.

0596g

JE24

The following Licensee Event Reports were submitted during the month of December 1985:

Licensee Event Report 50-219/85-020 - On November 11, 1985 Diesel Generator (DG) 1-2 was declared inoperable because of a failure in the electric governor actuator controller. Prior to this, DG 1-1 was removed from service for battery replacement. The root cause was attributed to an electronic component failure within the electric governor actuator. Corrective action consisted of replacing the electric governor actuator. The failed unit was sent to the manufacturer to identify the failed electronic component.

Licensee Event Report 50-219/85-021 - On November 19, 1985, the System I Average Power Range Monitor (APRM) flow converter trip setpoint was found to exceed both the as-found acceptance criteria of the surveillance and the Limiting Safety System Setpoint (LSSS) limit in the Oyster Creek Nuclear Generating Station technical specifications. The root cause was attributed to drifting of the trip setpoint in the electronic components of the flow converter unit. The immediate corrective action taken was to adjust the trip setpoint down to meet the acceptance criteria of the surveillance procedure and the technical specifications.

Licensee Event Report 50-219/85-022 - On November 20, 1985, Main Generator tripped causing a turbine trip and full reactor scram. Operators reset scram, but a Main Steam Isolation Valve (MSIV) closure scram occurred. Isolation Condensers were then used to control reactor pressure and initiate a shutdown. The post trip review concluded that operator ranging up the Intermediate Range Monitor (IRM) inadvertently went past range 9 to pick up range 10 contacts. This in conjunction with reactor pressure being less than 850 psig resulted in MSIV closure, and subsequent scram. The cause of the generator trip was a "B" phase differential current relay trip, caused by a failure in the secondary windings of the current transformer used to sense "B" phase current.

#### OPERATING DATA REPORT OPERATING STATUS

1. DOCKET: 50-219

2. REPORTING PERIOD: DECEMBER, 1985

3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699

4. LICENSED THERMAL POWER (MWt): 1930

5. NAMEPLATE RATING (GROSS MWe): 687.5 X 0.8 = 550

6. DESIGN ELECTRICAL RATING (NET MWe): 650

7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650

6. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620

9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE

10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): NONE

11. REASON FOR RESTRICTION, IF ANY: NONE

		MONTH	YEAR	CUMULATIVE
12.	REPORT PERIOD HRS	744.0	8760.0	140473.0
13.	HOURS RX CRITICAL	701.0	6819.5	92447.4
14.	RX RESERVE SHIDWN HRS	0.0	0.0	469.7
15.	HRS GENERATOR ON-LINE	684.0	6522.4	90059.1
16.	UT RESERVE SHIDWN HRS	0.0	753.1	755.8
17.	GROSS THERM ENER (MWH)	1291000	11615140	148877469
18.	GROSS ELEC ENER (MWH)	436510	3907690	50290685
19.	NET ELEC ENER (MWH)	419797	3746036	48308601
20.	UT SERVICE FACTOR	91.9	74.5	64.1
21.	UT AVAIL FACTOR	91.9	83.1	64.6
22.	UT CAP FACTOR (MDC NET)	91.0	69.0	55.5
23.	UT CAP FACTOR (DER NET)	86.8	65.8	52.9
24.	UT FORCED OUTAGE RATE	8.1	18.5	10.4
25.	FORCED OUTAGE HRS	60.0	1484.5	10435.6

<sup>26.</sup> SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION): REFUELING, APRIL 12, 1986, 6 MONTHS

<sup>27.</sup> IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME: N/A

# AVERAGE DAILY POWER LEVEL NET MWe

### MONTH DECEMBER, 1985

DAY	MW	DAY	MW
1.	626	16.	0
2.	629	17.	20
3.	629	18.	449
4.	627	19.	599
5.	625	20.	621
6.	625	21.	625
7.	622	22.	624
8.	625	23.	624
9.	626	24.	623
10.	625	25.	624
11.	623	26.	624
12.	624	27.	624
13.	622	28.	624
14.	624	29.	625
15.	196	30.	624
		31.	624

#### REFUELING INFORMATION - December, 1985

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: April 12, 1986

Scheduled date for restart following refueling: October 12, 1986

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Yes

Scheduled date(s) for submitting proposed licensing action and supporting information:

April, 1986

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

- General Electric Fuel Assemblies fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
- Exxon Fuel Assemblies no major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core = 560
(b) in the spent fuel storage pool = 1204
(c) in the dry storage = 26
(d) in temporary storage = 182

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present licensed capacity: 2600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Reracking of the fuel pool is in progress. Four out of ten (10) racks have been installed to date. When reracking is completed, discharge capacity to the spent fuel pool will be available until 1990 refurling outage.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

50-219 DOCKET NO. Oyster Creek UNIT NAME January 1986 DATE R. Baran **COMPLETED BY** 971-4640 TELEPHONE

REPORT MONTH December 1985

No.	Date	Type1	Duration (Hours)	Reason2	Method of Shutting Down Reactor3	Licensee Event Report #	System Code <sup>4</sup>	Component Code5	Cause & Corrective Action to Prevent Recurrence
43	12/15/85	F	60.0	A	3	N/A	ZZ	ZZZZZZ	Reactor Scram on high neutron flux due to pressure spike caused by closure of the turbine control valves Closure of the valves resulted from a loose connection to the position feedback circuitry.

F: Forced S: Scheduled Reason:

A-Equipment Failure (Explain) B-Maintenance of Test

C-Refueling

D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative

G-Operational Error (Explain) 11-Other (Explain)

3 Method:

1-Manual

2-Manual Scram.

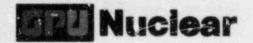
3-Automatic Scram.

4-Other (Explain)

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit 1 - Same Source

(9/77)



**GPU Nuclear Corporation** 

Post Office Box 388 Route 9 South Forked River, New Jersey 08731-0388 609 971-4000 Writer's Direct Dial Number:

January 13, 1986

Director
Office of Management Information
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station

Docket No. 50-219

Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Mr. Drew Holland at (609) 971-4643.

Very truly yours,

Reter B. Fiedler

Vice President and Director Oyster Creek

PBF:KB:dam(0170A) Enclosures

cc: Director (10)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dr. Thomas E. Murley, Administrator Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Pressia, PA 19406

Mr. Jack N. Donohew, r. U.S. Nuclear Regulatory Commission 7920 Norfolk Avenue, Phillips Bldg. Bethesda, MD 20014

NRC Resident Inspector Oyster Creek Nuclear Generating Station TE24