

ATTACHMENT 1

PROPOSED TECHNICAL SPECIFICATION 4.6.1.2a

VIRGINIA ELECTRIC AND POWER COMPANY

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CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of:
 1. $\leq L_a$, 0.1 percent by weight of the containment air per 24 hours at $P_a \geq 40.6$ psig, or
- b. A combined leakage rate of $\leq 0.60 L_a$ for all penetrations and valves subject to Type B and C tests, when^a pressurized to $P_a \geq 40.6$ psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$ or (o) with the measured combined leakage rate for all penetrations and^a valves subject to Type B and C tests exceeding $0.60 L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972:

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at $P_a \geq 40.6$ psig during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.*

*The third test of the first 10-year service period shall be conducted during the 1989 Refueling/10-Year ISI Outage.

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of:
 1. Less than or equal to L_a , 0.1 percent by weight of the containment air per 24 hours at P_a , greater than or equal to 40.6 psig, or
- b. A combined leakage rate of less than or equal to $0.60 L_a$ for all penetrations and valves subject to Type B and C tests, when pressurized to P_a , greater than or equal to 40.6 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$ or (b) with the measured combined leakage rate for all penetrations and valves subject to Type B and C tests exceeding $0.60 L_a$, restore the overall integrated leakage rate to less than $0.75 L_a$ and the combined leakage rate for all penetrations subject to Type B and C tests to less than or equal to $0.60 L_a$ prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972:

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at P_a greater than or equal to 40.6 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.*

*The third test of the first 10-year service period shall be conducted during the 1989 Refueling/10-Year ISI Outage.

ATTACHMENT 2

SAFETY EVALUATION

VIRGINIA ELECTRIC AND POWER COMPANY

DISCUSSION

This amendment will modify the Unit 1 and 2 Technical Specifications to permit conducting the third Type A test of the first 10-year service period during the 1989 Refueling/10-Year ISI Outage. Currently, because of the Technical Specification requirements to conduct Type A tests at a 40 ± 10 month frequency, the third Type A test would be due on or before November 11, 1988 for Unit 1 and on or before December 14, 1988 for Unit 2. The Technical Specifications also specify that the "third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection." The Unit 1 outage is currently scheduled to begin in April 1989 and the Unit 2 outage is currently scheduled to begin in February 1989.

The second interval Type A test for Unit 1 was completed on September 11, 1984. This test showed a total containment integrated leakage rate of approximately 33% of La (43.5% of 0.75 La), which included leakage attributable to penetrations and valves as measured by Types B and C tests of approximately 16.5% of La. The last measured Types B and C leakage rates were less than 17% of La (27.1% of 0.60 La). These leakage rates indicate that the Unit 1 containment integrity is adequate.

The second interval Type A test for Unit 2 was completed on October 14, 1984. This test showed a leakage rate of approximately 70% of La (92.3% of 0.75 La) which included leakage attributable to penetrations and valves as measured by Types B and C tests of approximately 10.6% of La. The last measured Types B and C leakage rates were less than 10% of La (15.4% of 0.60 La). Therefore, Unit 2 containment integrity is adequate.

BASIS FOR NO SIGNIFICANT HAZARDS DETERMINATION

The proposed extension of the surveillance interval for the third Type A test does not involve a significant increase in the probability or consequences of an accident previously evaluated. The last measured Types A, B, and C leakage rates indicate that Unit 1 and 2 containment integrity is adequate. In addition, leakage from containment penetrations and valves, including air locks, is measured in accordance with Technical Specifications 3/4.6.1.2 and 3/4.6.1.3 whenever changes or activities occur (e.g., valve maintenance or modification, containment entries) which may affect leakage rate. Thus, the combined leakage of penetrations subject to Types B and C tests will continue to be maintained within Technical Specifications' limits. Therefore the proposed extension in the surveillance interval for the Type A test will not result in a significant increase in the probability or consequences of an accident previously evaluated.

The proposed extension of the surveillance interval does not create the possibility of a new or different kind of accident from any accident previously evaluated. The change does not impact the design basis of the containment and does not modify the response of the containment during a design basis accident.

The proposed extension of the surveillance interval does not involve a significant reduction in the margin of safety. The 1984 Type A test results indicate that the containment integrity is adequate. In addition, leakage from containment penetrations and valves, including air locks, is measured in accordance with Technical Specifications 3/4.6.1.2 and 3/4.6.1.3 whenever changes or activities occur (e.g., valve maintenance or modification, containment entries) which may affect leakage rate. Thus, the combined leakage of penetrations subject to Types B and C tests will continue to be maintained within Technical Specifications' limits. Therefore the proposed extension in the surveillance interval for the Type A test will not result in a significant reduction in the margin of safety.

Therefore, pursuant to 10CFR50.92, based on the above considerations, it has been determined that this change does not involve a significant safety hazards consideration.