# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 86-354
UNIT Hope Creek
DATE 9/15/88
COMPLETED BY H. Jensen
TELEPHONE (609) 339-5261

MONTH August 1988

| DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|---|---|---|---|
| 1 | 989 | 17 | 995 |
| 2 | 983 | 18 | 1014 |
| 3 | 980 | 19 | 996 |
| 4 | 990 | 20 | 822 |
| 5 | 950 | 21 | 1011 |
| 6 | 533 | 22 | 1009 |
| 7 | 570 | 23 | 1014 |
| 8 | 997 | 24 | 1005 |
| 9 | 867 | 25 | 992 |
| 10 | 999 | 26 | 744 |
| 11 | 1003 | 27 | 0 |
| 12 | 972 | 28 | 0 |
| 13 | 996 | 29 | 454 |
| 14 | 989 | 30 | 1010 |
| 15 | 969 | 31 | 993 |
| 16 | 1001 | | |

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# OPERATING DATA REPORT

DOCKET NO. 86-354
UNIT Hope Creek
DATE 9/15/88
COMPLETED BY H. Jensen
TELEPHONE (609) 339-5261

OPERATING STATUS

1. REPORTING PERIOD August 1988 GROSS HOURS IN REPORTING PERIOD 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt) 3293
   MAX. DEPEND. CAPACITY (MWe-Net) 1067 (1)
   DESIGN ELECTRICAL RATING (MWe-Net) 1067
   NAMEPLATE RATING (GROSS MWe) 1170

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net) None

4. REASONS FOR RESTRICTION (IF ANY)

| | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|---|---|---|---|
| 5. | NO. OF HOURS REACTOR WAS CRITICAL | 695.2 | 4,375.3 | 12,333.4 |
| 6. | REACTOR RESERVE SHUTDOWN HOURS | 0 | 0 | 0 |
| 7. | HOURS GENERATOR ON LINE | 681.8 | 4,205.8 | 11,950.9 |
| 8. | UNIT RESERVE SHUTDOWN HOURS | 0 | 0 | 0 |
| 9. | GROSS THERMAL ENERGY GENERATED (MWH) | 2,161,396 | 13,605,653 | 37,414,220 |
| 10. | GROSS ELECTRICAL ENERGY GENERATED (MWH) | 691,963 | 4,464,357 | 12,376,055 |
| 11. | NET ELECTRICAL ENERGY GENERATED (MWH) | 660,565 | 4,259,367 | 11,824,405 |
| 12. | REACTOR SERVICE FACTOR | 93.4 | 74.7 | 82.1 |
| 13. | REACTOR AVAILABILITY FACTOR | 93.4 | 74.7 | 82.1 |
| 14. | UNIT SERVICE FACTOR | 91.6 | 71.8 | 80.2 |
| 15. | UNIT AVAILABILITY FACTOR | 91.6 | 71.8 | 80.2 |
| 16. | UNIT CAPACITY FACTOR (Using MDC) | 83.2 | 68.2 | 74.4 |
| 17. | UNIT CAPACITY FACTOR (Using Design MWe) | 83.2 | 68.2 | 74.4 |
| 18. | UNIT FORCED OUTAGE RATE | 8.4 | 3.5 | 7.1 |

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, & DURATION):
    1/14/89, mid-cycle, 21 days

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
    N/A

(1) August 1987 data is under management review.

OPERATING DATA REPORT

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 86-354

UNIT Hope Creek

DATE 9/15/88

COMPLETED BY H. Jensen

REPORT MONTH August, 1988 TELEPHONE (609) 339-5261

| NO. | DATE | F S | TYPE FORCED SCHEDULED | DURATION (HOURS) | REASON (1) | METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2) | CORRECTIVE ACTION/ COMMENTS |
|---|---|---|---|---|---|---|---|
| 8 | 8/6 | | F | 0 | A | 5 | 'A' REACTOR RECIRCULATION PUMP PROBLEMS |
| 9 | 8/26 | | F | 62.2 | A | 3 | AUTOMATIC REACTOR SCRAM DUE TO A TURBINE TRIP DURING THE PERFORMANCE OF A WEEKLY MAIN TURBINE FUNCTIONAL TEST. LER 88-022 |

# REFUELING INFORMATION

COMPLETED BY: Chris Brennan

DOCKET NO.: 50-354
UNIT NAME: Hope Creek Unit 1
DATE: 9/15/88
TELEPHONE: (609) 935-

Month August 1988

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: 09-22-89

3. Scheduled date for restart following refueling:

   11-07-89

4. A) Will Technical Specification changes or other license amendments be required?
   YES X NO _____

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?

   YES _____ NO X

   If no, when is it scheduled? 5-07-89

5. Scheduled date(s) for submitting proposed licensing action:
   6-07-89

6. Important licensing considerations associated with refueling:
   Information not presently available

7. Number of Fuel Assemblies:
   A) Incore 764
   B) In Spent Fuel Storage 232

8. Present licensed spent fuel storage capacity: 1108

   Future spent fuel storage capacity: 4006

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: 11-07-89

HOPE CREEK GENERATING STATION
MONTHLY OPERATING SUMMARY
AUGUST 1988

The unit entered the month of August at approximately 100% power. Power was reduced on August 6 due to problems with the "A" Reactor Recirculation Pump. The pump was repaired and power was restored to 100%. At 6:25 PM on August 26, the reactor scrammed due to a turbine trip during the performance of a weekly Main Turbine Functional Test. The plant had been on-line for 112 consecutive days. The unit was returned to the grid at 7:30 AM on August 29.

R-014
RAR:tlb

# SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS

# FOR THE HOPE CREEK GENERATING STATION

AUGUST 1988

The following Design Change Packages (DCPs) have been evaluated to determine:

1) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2) if a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3) if the margin of safety as defined in the basis for any technical specification is reduced.

None of the DCPs created a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These DCPs did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

| DCP | Description of Design Change Package |
|---|---|
| 4EC-1082/02 | This DCP incorporated a fix for a human engineering deficiency that was reported during the Control Room Design Review. The Off-Gas section of the Main Control Panel contained unnecessary components. The unnecessary lenses were removed and replaced with a single blank, which was weighted to comply with seismic considerations. |
| 4HC-0074/03 | This DCP upgraded the firmware and software in the Liquid Radwaste Radiation Monitoring System. The upgrades included a logic table defining monitor operate status, a digital input representing flow, a software time delay to monitor operate status change, a digital reset to reset monitor operate status, and a manual override switch to override a system isolation signal under false indication. These changes eliminated the need for operator aids, TMRs, and operator intervention. |
| 4HC-0089 | This DCP provided for the proper routing of conduit associated with the Crystallizer Vapor Body Level Control instruments. The DCP shortens the distance from the transmitters to the level probes to obtain a more precise instrument reading. This DCP also installed new scales in the Radwaste Control Room. |
| 4HM-0004 | This DCP replaced temperature switches in the Turbine Building Chilled Water System. The original switches did not have an adjustable deadband. The new switches will also support a setpoint change to prevent excessive chiller recycling. |
| 4HM-0109 | This DCP relocated the Reactor Recirculation Motor/Generator Set Fluid Coupler Pressure Differential Switches to eliminate oil from leaking into the switch housing. It also re-tubed the sensing lines to maintain a positive downward slope from the instrument to the fluid drive. Additionally, this DCP changed the associated computer points from "close to alarm" to "open to alarm". |
| 4HM-0236 | This DCP deleted the bridge third speed of the 7 1/2 ton Radwaste crane. The speed was deleted because it was determined to be too fast for the available space. The DCP also corrected a drawing error dealing with the wiring for the Monorail Hoist Stop Limit Switch. |

| DCP | Description of Design Change Package |
|---|---|
| 4HM-0299 | This DCP raised the alarm setpoints on 228 heat trace controllers from a 10 deviation to a 15 deviation. Raising the setpoints eliminated nuisance alarms and ensured that future "Heat Trace System Trouble" alarms would be more meaningful. |
| 4HM-0312 | This DCP was a change-out DCP that replaced the High Voltage Power Supply in the In-line Duct Monitors and the Drywell Leak Detection Radiation Monitoring System Noble Gas Monitor. The new power supplies are an improved design that provide additional reliability by eliminating the failure mechanisms due to heat buildup. |
| 4HM-0315 | This DCP installed Secondary Turbine Vibration Instrumentation on existing shaft rider detectors for diagnostic and monitoring purposes. The information obtained through the monitoring will be used to improve the reliability of the Main Turbine. |
| 4HM-0369 | This DCP was a paper change only DCP. It corrected discrepancies between vendor documents, plant design documents, and Instrument Calibration Data Cards. The documentation now reflects current plant configuration for the Chilled Water system. |
| 4HM-0389 | This DCP replaced the steam trap in a section of pipe that is isolated on a High Pressure Coolant Injection start signal. The old steam trap is no longer being manufactured, the new steam trap is equally qualified for the application. |

The following Temporary Modification Requests (TMRs) have been evaluated to determine:

1) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2) if a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3) if the margin of safety as defined in the basis for any technical specification is reduced.

None of the TMRs created a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These TMRs did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

| Safety Evaluation | Description of Temporary Modification Request (TMR) |
|---|---|
| 88-0101 | This TMR raised the radial vibration alarm setpoint on the 'B' Secondary Condensate Pump. This change allows the retention of the ability to detect vibration changes and the elimination of nuisance annunciator alarms in the Control Room, without affecting the associated danger setpoint. |
| 88-0104 | This TMR added a jumper across contacts on pressure switches to operate the Turbine Building Chillers until data for a design change to increase the setpoint can be developed. Each chiller is equipped with two pressure switches. This TMR jumpers out the pressure switches with the lower setpoint. This TMR is required due to higher than normal Chiller Condenser pressure caused by abnormally hot weather. |
| 88-0106 | This TMR removed the operator from a valve in the Turbine Auxiliaries Cooling System Demineralizers and gagged the valve open. This allows the Demineralizer to be returned to service to maintain the systems water chemistry within specifications. |

PSE&G

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Nuclear Department

September 15, 1988

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATING STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for August are being forwarded to you. In addition, the summary of changes, tests, and experiments for August 1988 is included pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

J. Hagan

J. Hagan
General Manager -
Hope Creek Operations

RAR RAR:tlb
Attachment

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