



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY
OLD DOMINION ELECTRIC COOPERATIVE
DOCKET NO. 50-338
NORTH ANNA POWER STATION, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 103
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated March 18, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

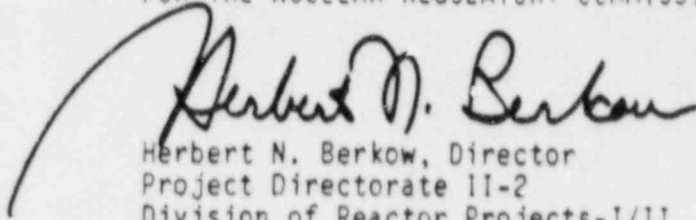
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 103, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 14 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 26, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 103

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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TABLE 6.2-1^a

MINIMUM SHIFT CREW COMPOSITION

Total Staffing Requirements for Station Operation

With Either or Both Units in Mode 1, 2, 3 or 4

POSITION - NUMBER - CONDITIONS

<u>SS</u>	-	ONE	(Shift Supervisor may fulfill duties for both units).
<u>SRO</u>	-	ONE	(If ONE unit is in MODE 5, 6 OR DEFUELED, Senior Reactor Operator is assigned to the Unit in MODE 1, 2, 3 or 4).
<u>RO</u>	-	THREE	(ONE Reactor Operator is assigned to each unit <u>PLUS</u> one is shared by both units).
<u>AO</u>	-	FOUR	(TWO Auxiliary Operators are assigned to each unit).
<u>STA</u>	-	ONE	(Shift Technical Advisor may fulfill duties for both units).

With Both Units in Mode 5 or 6 (or DEFUELED)

POSITION - NUMBER - CONDITIONS

<u>SS</u>	-	ONE	(Shift Supervisor may fulfill duties for both units).
<u>SRO</u>	-	NONE	
<u>RO</u>	-	TWO	(ONE Reactor Operator is assigned to each unit).
<u>AO</u>	-	TWO	(ONE Auxiliary Operator is assigned to each unit).
<u>STA</u>	-	ONE	(Shift Technical Advisor may fulfill duties for both units).

a - This Table and Table 6.2.1 of Unit 2 Technical Specifications represent Total Station Staffing and ARE NOT ADDITIVE.

ADMINISTRATIVE CONTROLS (Continued)

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Regional Administrator, Region II, within the time period specified for each report. These reports shall be submitted pursuant to the requirement of the applicable specification:

- a. Inservice Inspection Reviews, Specification 4.0.5, shall be reported within 90 days of completion.
- b. MODERATOR TEMPERATURE COEFFICIENT. Specification 3.1.1.4.
- c. RADIATION MONITORING INSTRUMENTATION. Specification 3.3.3.1, TABLE 3.3-6, Action 35.
- d. SEISMIC INSTRUMENTATION. Specifications 3.3.3.3 and 4.3.3.3.2.
- e. METEOROLOGICAL INSTRUMENTATION. Specification 3.3.3.4.
- f. FIRE DETECTION INSTRUMENTATION. Specification 3.3.3.7.
- g. LOOSE PARTS MONITORING SYSTEMS. Specification 3.3.3.9.
- h. REACTOR COOLANT SYSTEM SPECIFIC ACTIVITY. Specification 3.4.8.
- i. OVERPRESSURE PROTECTION SYSTEMS. Specification 3.4.9.3.
- j. EMERGENCY CORE COOLING SYSTEMS. Specification 3.5.2 and 3.5.3.
- k. SETTLEMENT OF CLASS 1 STRUCTURES. Specification 3.7.12.
- l. GROUND WATER LEVEL - SERVICE WATER RESERVOIR. Specification 3.7.13.
- m. FIRE SUPPRESSION SYSTEMS. Specifications 3.7.14.1, 3.7.14.2, 3.7.14.3, 3.7.14.4, and 3.7.14.6.
- n. RADIOACTIVE EFFLUENTS. Specifications 3.11.1.2, 3.11.1.3, 3.11.2.2, 3.11.2.3, 3.11.2.4 and 3.11.4.
- o. RADIOLOGICAL ENVIRONMENTAL MONITORING. Specification 3.12.1.b.
- p. SEALED SOURCE CONTAMINATION. Specification 4.7.11.1.3.
- q. REACTOR COOLANT SYSTEM STRUCTURAL INTEGRITY. Specification 4.4.10. For any abnormal degradation of the structural integrity of the reactor vessel or the Reactor Coolant System pressure boundary detected during the performance of Specification 4.4.10, an initial report shall be submitted within 10 days after detection and a detailed report submitted within 90 days after the completion of Specification 4.4.10.

ADMINISTRATIVE CONTROLS

- r. CONTAINMENT STRUCTURAL INTEGRITY. Specification 4.6.1.6. For any abnormal degradation of the containment structure detected during the performance of Specification 4.6.1.6, an initial report shall be submitted within 10 days after the completion of Specification 4.6.1.6. A final report, which includes (1) a description of the condition of the liner plate and concrete, (2) inspection procedure, (3) the tolerance on cracking, and (4) the corrective actions taken, shall be submitted within 90 days after the completion of Specification 4.6.1.6.

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE EVENTS and Special Reports.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of changes made to Operating Procedures.
- f. Records of radioactive shipments.
- g. Records of sealed source leak tests and results.
- h. Records of annual physical inventory of all sealed source material of record.
- i. Records of the annual audit of the Station Emergency Plan and implementing procedures.
- j. Records of the annual audit of the Station Security Plan and implementation procedures.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

ADMINISTRATIVE CONTROLS

6.13 DELETED

6.14 PROCESS CONTROL PROGRAM (PCP)

6.14.1 Licensee initiated changes to the PCP:

1. Shall be submitted to the Commission in Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:
 - a. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information;
 - b. A determination that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and
 - c. Documentation of the fact that the change has been reviewed and found acceptable by the SNSOC.
2. Shall become effective upon review and acceptance by the SNSOC.

ADMINISTRATIVE CONTROLS

6.15 OFFSITE DOSE CALCULATION MANUAL (ODCM)

6.15.1 The ODCM shall be approved by the Commission prior to implementation.

6.15.2 Licensee initiated changes to the ODCM:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made effective. This submittal shall contain:
 - a. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered and provided with an approval and date box, together with appropriate analyses or evaluations justifying the change(s);
 - b. A determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and
 - c. Documentation of the fact that the change has been reviewed and found acceptable by the SNSOC.
2. Shall become effective upon review and acceptance by the SNSOC.

6.16 MAJOR CHANGES TO RADIOACTIVE SOLID WASTE TREATMENT SYSTEMS*

6.16.1 Licensee initiated major changes to the radioactive solid waste systems:

1. Shall be reported to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the evaluation was reviewed by SNSOC. The discussion of each change shall contain:
 - a. A summary of the evaluation that led to the determination that the change could be made in accordance with 10 CFR Part 50.59.
 - b. Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information;
 - c. A detailed description of the equipment, components and processes involved and the interfaces with other plant systems;

*Licensees may chose to submit the information called for in this Specification as part of the annual FSAR update.



UNITED STATES
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VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated March 18, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

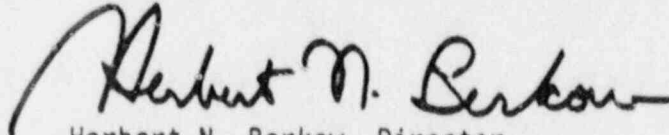
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 90 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. In addition, Facility Operating License NPF-7 is amended by deleting License Conditions 4.a, 4.b, 4.d, and 4.e.
4. This license amendment is effective as of the date of issuance and shall be implemented within 14 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 26, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 90
TO FACILITY OPERATING LICENSE NO. NPF-7
DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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TABLE 6.2-1^a

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Total Staffing Requirements for Station Operation

With Either or Both Units in Mode 1, 2, 3 or 4

POSITION - NUMBER - CONDITIONS

<u>SS</u>	-	ONE	(Shift Supervisor may fulfill duties for both units).
<u>SRO</u>	-	ONE	(If ONE unit is in MODE 5, 6 OR DEFUELED, Senior Reactor Operator is assigned to the Unit in MODE 1, 2, 3 or 4).
<u>RO</u>	-	THREE	(ONE Reactor Operator is assigned to each unit <u>PLUS</u> one is shared by both units).
<u>AO</u>	-	FOUR	(TWO Auxiliary Operators are assigned to each unit).
<u>STA</u>	-	ONE	(Shift Technical Advisor may fulfill duties for both units).

With Both Units in Mode 5 or 6 (or DEFUELED)

POSITION - NUMBER - CONDITIONS

<u>SS</u>	-	ONE	(Shift Supervisor may fulfill duties for both units).
<u>SRO</u>	-	NONE	
<u>RO</u>	-	TWO	(ONE Reactor Operator is assigned to each unit).
<u>AO</u>	-	TWO	(ONE Auxiliary Operator is assigned to each unit).
<u>STA</u>	-	ONE	(Shift Technical Advisor may fulfill duties for both units).

a - This Table and Table 6.2.1 of Unit 1 Technical Specifications represent Total Station Staffing and ARE NOT ADDITIVE.

ADMINISTRATIVE CONTROLS

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Regional Administrator, Region II, within the time period specified for each report. These reports shall be submitted pursuant to the requirement of the applicable specification:

- a. Inservice Inspection Reviews, Specification 4.0.5, shall be reported within 90 days of completion.
- b. MODERATOR TEMPERATURE COEFFICIENT. Specification 3.1.1.4.
- c. FIRE DETECTION INSTRUMENTATION. Specification 3.3.3.7.
- d. RADIATION MONITORING INSTRUMENTATION. Specification 3.3.3.1, TABLE 3.3-6 Action 35.
- e. REACTOR COOLANT SYSTEM SPECIFIC ACTIVITY. Specification 3.4.8.
- f. OVERPRESSURE PROTECTION SYSTEMS. Specification 3.4.9.3.
- g. EMERGENCY CORE COOLING SYSTEMS. Specification 3.5.2 and 3.5.3.
- h. SETTLEMENT OF CLASS 1 STRUCTURES. Specification 3.7.12.
- i. GROUND WATER LEVEL - SERVICE WATER RESERVOIR. Specification 3.7.13.
- j. FIRE SUPPRESSION SYSTEMS. Specifications 3.7.14.1, 3.7.14.2, 3.7.14.3, 3.7.14.4, 3.7.14.5 and 3.7.14.6.
- k. PENETRATION FIRE BARRIERS. Specification 3.7.15.
- l. RADIOACTIVE EFFLUENTS. Specifications 3.11.1.2, 3.11.1.3, 3.11.2.2, 3.11.2.3, 3.11.2.4 and 3.11.4.
- m. RADIOLOGICAL ENVIRONMENTAL MONITORING. Specification 3.12.1.b.
- n. SEALED SOURCE CONTAMINATION. Specification 4.7.11.1.3.
- o. REACTOR COOLANT SYSTEM STRUCTURAL INTEGRITY. Specification 4.4.10. For any abnormal degradation of the structural integrity of the reactor vessel or the Reactor Coolant System pressure boundary detected during the performance of Specification 4.4.10, an initial report shall be submitted within 10 days after detection and a detailed report submitted within 90 days after the completion of Specification 4.4.10.
- p. CONTAINMENT STRUCTURAL INTEGRITY. Specification 4.6.1.6. For any abnormal degradation of the containment structure detected during the performance of Specification 4.6.1.6, an initial report shall be submitted within 10 days after completion of Specification 4.6.1.6. A final report, which includes (1) a description of the condition of the liner plate and concrete, (2) inspection procedure, (3) the tolerance on cracking, and (4) the corrective actions taken, shall be submitted within 90 days after the completion of Specification 4.6.1.6.

ADMINISTRATIVE CONTROLS (Continued)

6.10 RECORD RETENTION

In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. All REPORTABLE EVENTS and Special Reports.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of changes made to Operating Procedures.
- f. Records of radioactive shipments.
- g. Records of sealed source leak tests and results.
- h. Records of annual physical inventory of all sealed source material of record.
- i. Records of the annual audit of the Station Emergency Plan and implementing procedures.
- j. Records of the annual audit of the Station Security Plan and implementing procedures.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material release to the environs.
- f. Records of transient or operational cycles for those facility components identified in Table 5.7-1.