

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 46 License No. NPF-35

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc., (licensees) dated May 26, 1988, as supplemented May 27, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations a. set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8806070124 880601 PDR ADOCK 05000413 PDR Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 46 , are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment was effective as of May 27, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

D. B. Matthews for

Gus C. Lainas, Assistant Director for Region II Reactors Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

Attachment: Technical Specification Changes

Date of Issuance: June 1, 1988

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OFFICIAL RECORD COPY

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ADR2:DRPR GCLainas



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAMBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39 License No. NPF-52

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency, (licensees) dated May 26, 1988, as supplemented May 27, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 39, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

This license amendment was effective as of May 27, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

D. B. Matthews for

Gus C. Lainas, Assistant Director for Region II Reactors Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

Attachment:

Technical Specification Changes

Date of Issuance: June 1, 1988

OFFICIAL RECORD COPY

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ATTACHMENT TO LICENSE AMENDMENT NO. 46

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND

TO LICENSE AMENDMENT NO. 39

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Amended Page

Overleaf Page

3/4 3-66

3/4 3-65

INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.6 The accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Total Number of Channels shown in Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 7 days, or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels except the unit vent-high-high range area monitor, the steam relief valve exhaust radiation monitor, the containment atmosphere-high range radiation monitor, and the reactor coolant radiation level less than the Minimum Channels OPERABLE requirements of Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT STAMDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- c. With the number of OPERABLE Channels for the unit vent-high-high range area monitor, or the steam relief valve exhaust radiation monitor, or the containment atmosphere-high range radiation monitor, or the reactor coolant radiation level less than required by the Minimum Channels OPERABLE requirements, initiate an alternate method of monitoring the appropriate parameter(s) within 72 hours, and either restore the inoperable channel(s) to OPERABLE status within 7 days or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days that provides actions taken, cause of the inoperability, and the plans and schedule for restoring the channels to OPERABLE status.
- d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.6 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

TABLE 3.3-10

ACCIDENT MONITORING INSTRUMENTATION

BA - UNITS 1 & 2	INSTRUMENT		TOTAL NO. OF CHANNELS	MINIMUM CHANNELS OPERABLE
	1.	Containment Pressure	2	1
	2.	Reactor Coolant Outlet Temperature - THOT (Wide Range)	2	1
	3.	Reactor Coolant Inlet Temperature - T _{COLD} (Wide Range)	2	1
	4.	Reactor Coolant Pressure - Wide Range	2	1
	5.	Prescurizer Water Level	2	1
3/4	6.	Steam Line Pressure	2/steam generato	r 1/steam generator
3-66	7.	Steam Generator Water Level - Narrow Range	2/steam generato	r 1/steam generator
	8.	Refueling Water Storage Tank Water Level	2	1
	9.	Auxiliary Feedwater Flow Rate	2/steam generato	r 1/steam generator
Amer	10.	Reactor Coolant System Subcooling Margin Monitor	1	1
	11.	PORV Position Indicator*	2/Valve	1/Valve
Amendment	12.	PORV Block Valve Position Indicator**		
Z Z		a. Unit 1 b. Unit 2	2/Valve 1/Valve	1/Valve 1/Valve
39	13.	Pressurizer Safety Valve Position Indicator	1/Valve	1/Valve
(Unit	14.	Containment Sump Water Level (Wide Range)	2	1