



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 93
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Commonwealth Edison Company (the licensee) dated October 2, 1984 and October 29, 1985, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-29 is hereby amended to read as follows:

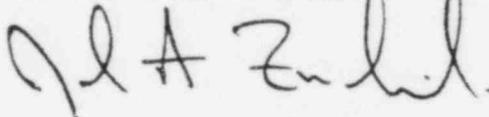
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 93, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 13, 1986.

ATTACHMENT TO LICENSE AMENDMENT NO. 93

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

Fig 3.5-1 (Sheet 1 of 4)

Fig 3.5-1 (Sheet 2 of 4)

Fig 3.5-1 (Sheet 3 of 4)

Fig 3.5-1 (Sheet 4 of 4)

5.0-1

INSERT

Fig 3.5-1 (Sheet 1 of 4)

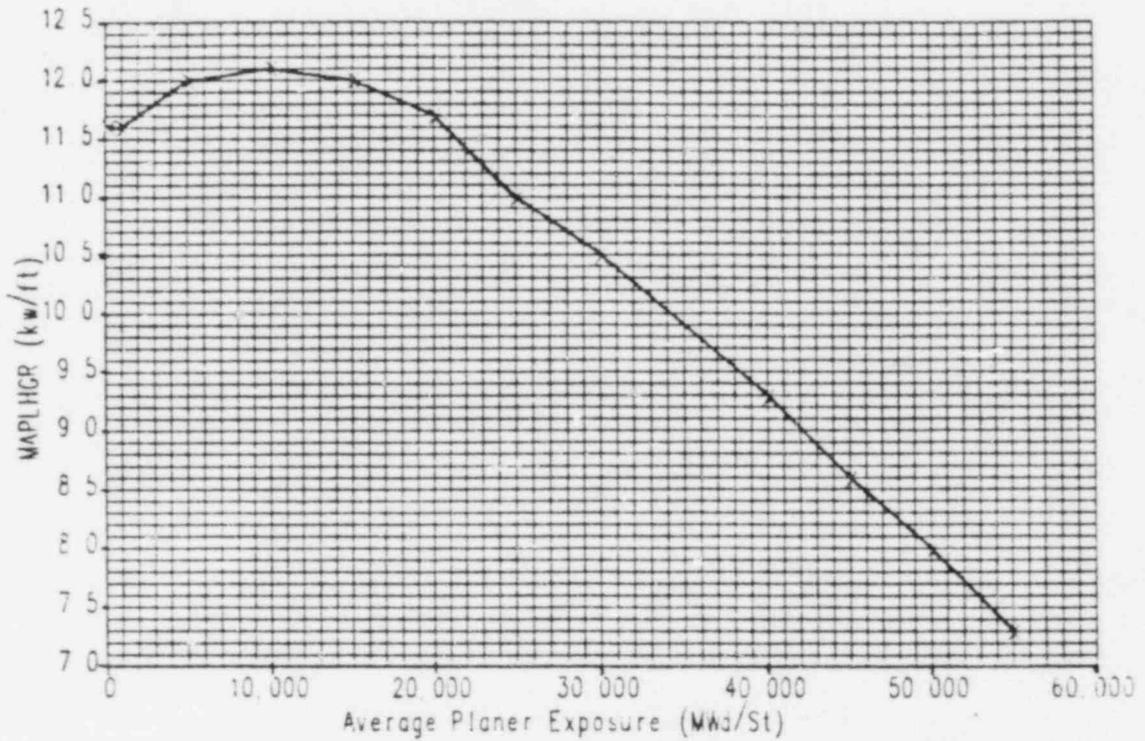
Fig 3.5-1 (Sheet 2 of 4)

Fig 3.5-1 (Sheet 3 of 4)

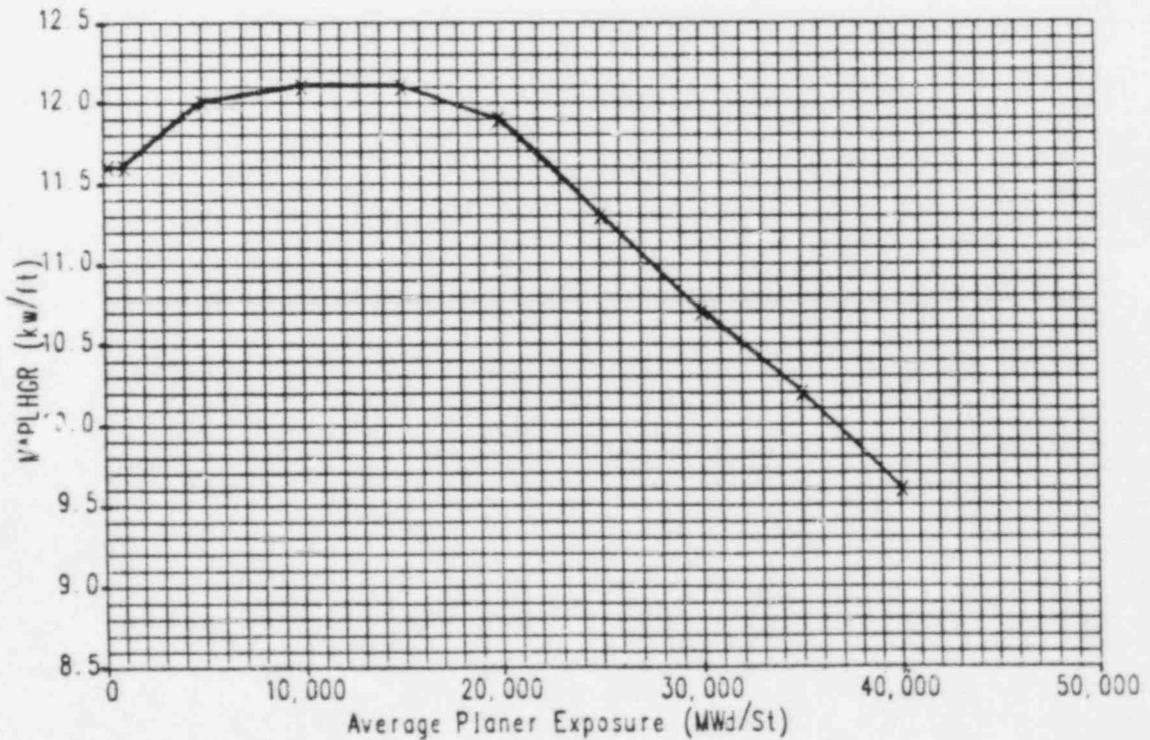
Fig 3.5-1 (Sheet 4 of 4)

5.0-1

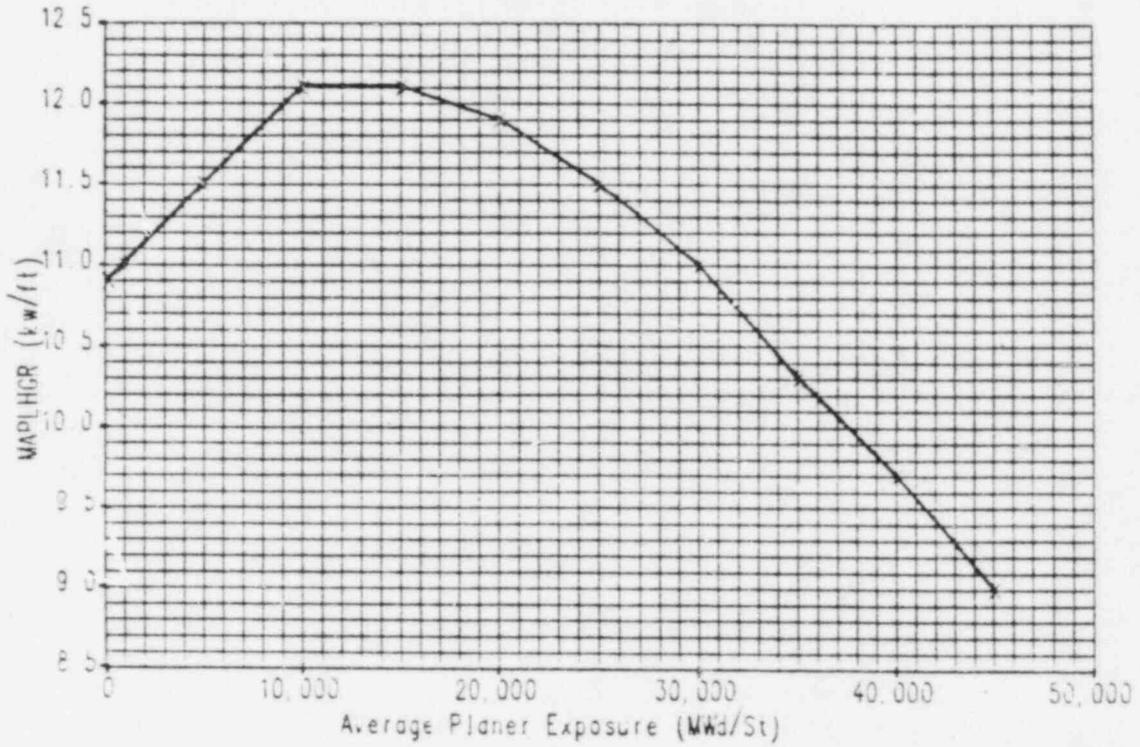
MAPLHGR Vs. Average Planer Exposure
Fuel Type Bärrier LTA



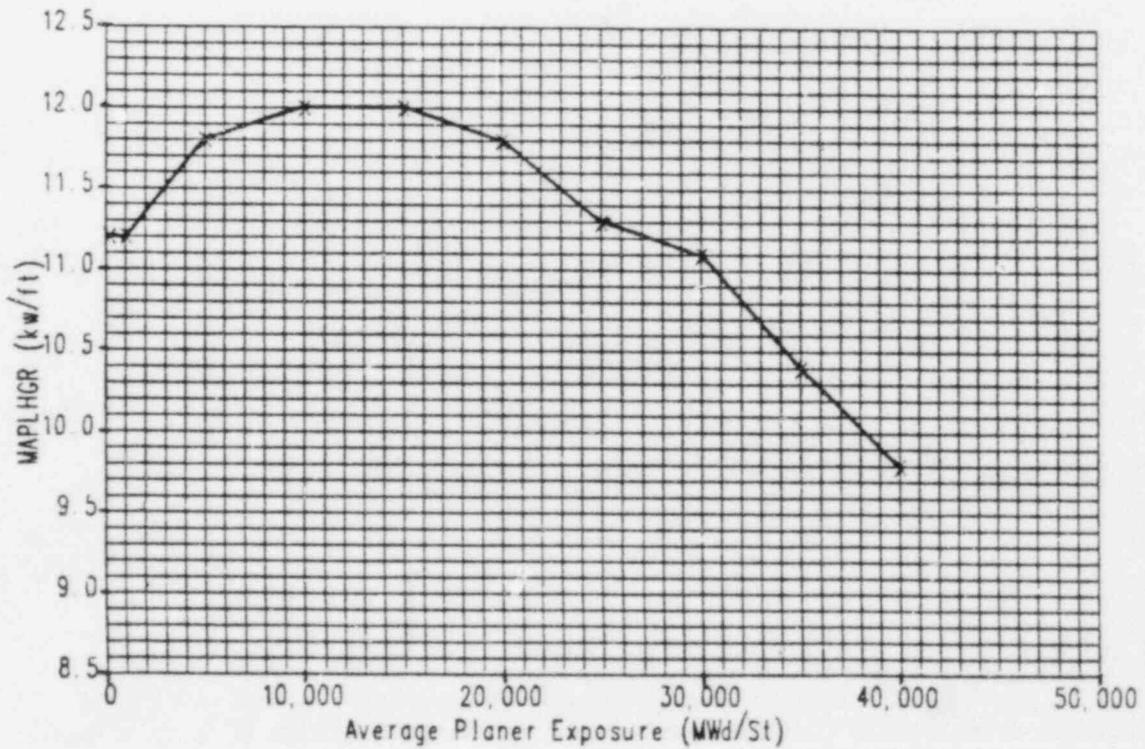
MAPLHGR Vs. Average Planer Exposure
Fuel Types P8DRB265L/P8DGB265L



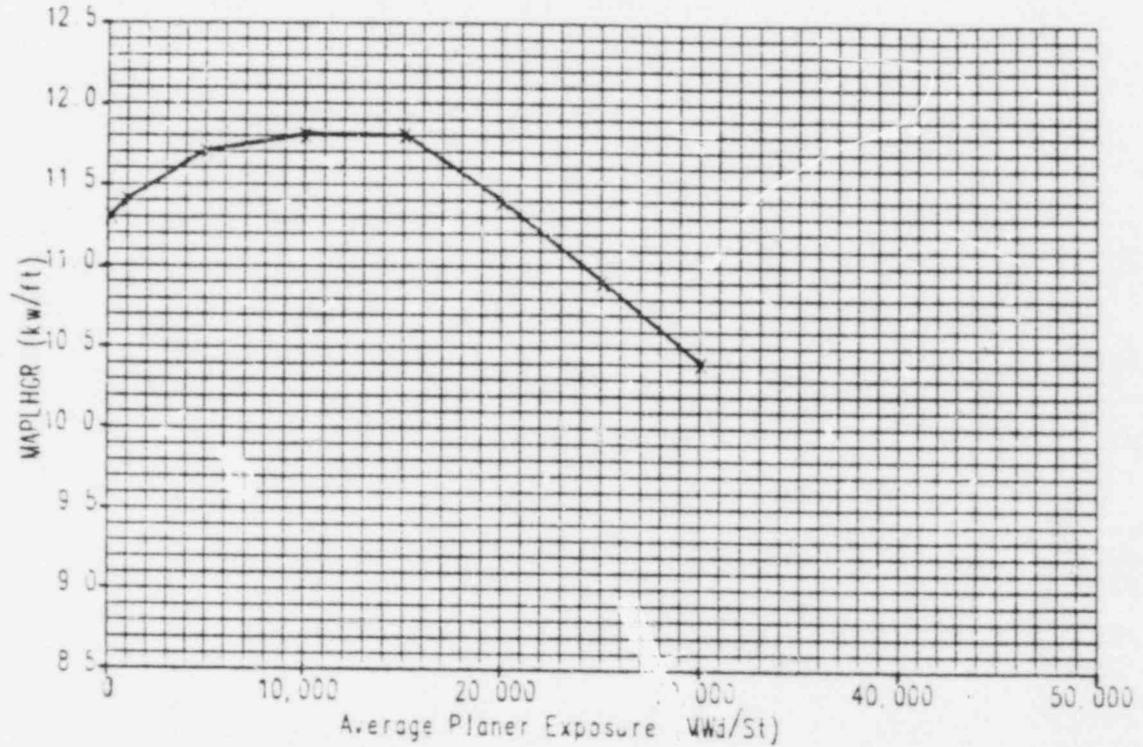
MAPLHGR Vs. Average Planer Exposure
Fuel Type BP8DRB299



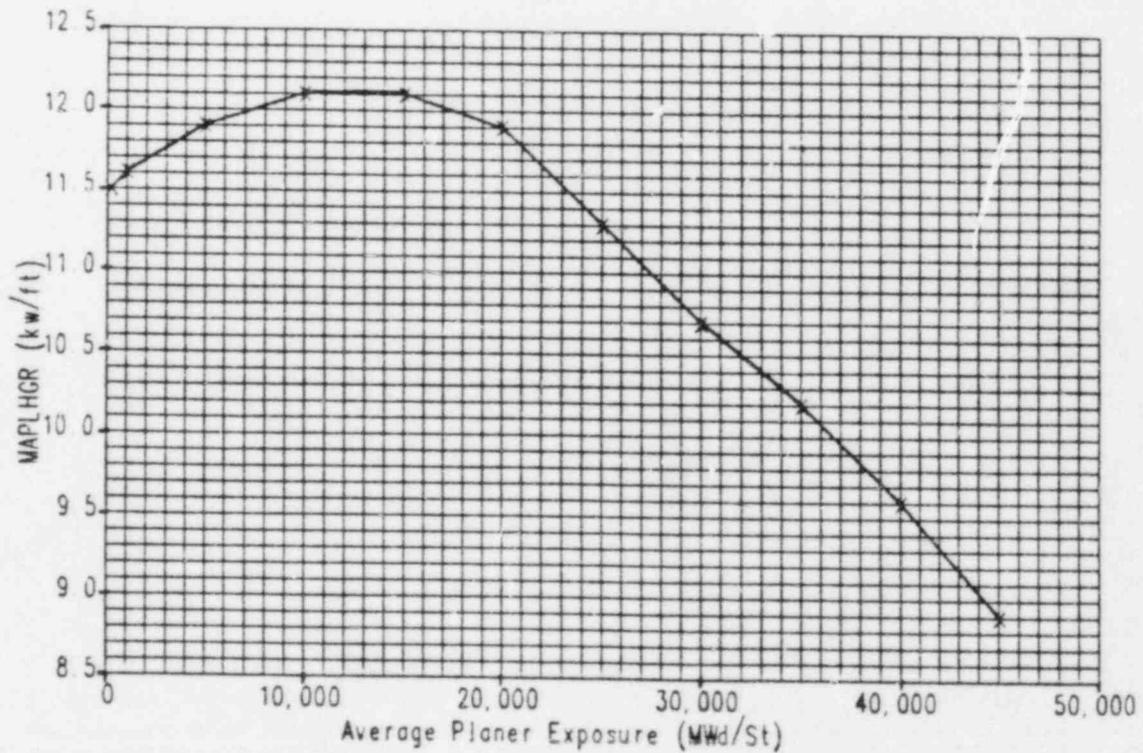
MAPLHGR Vs. Average Planer Exposure
Fuel Type P8DRB282



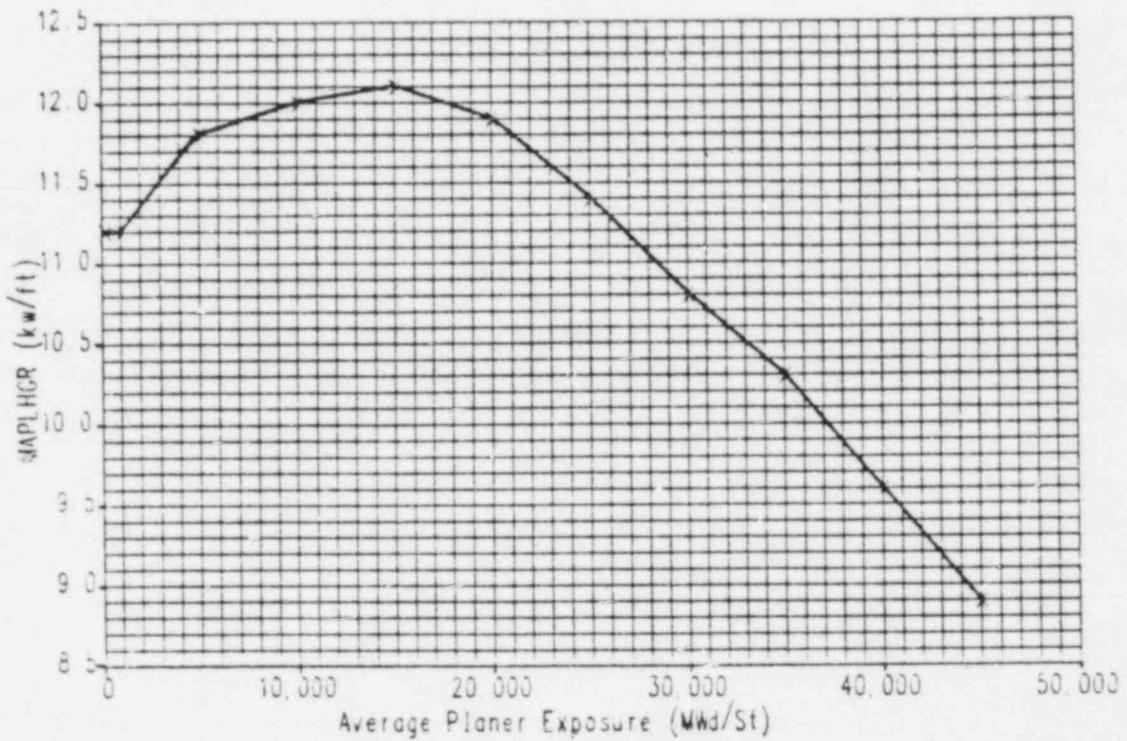
MAPLHGR Vs. Average Planer Exposure
Fuel Type P8DRB239



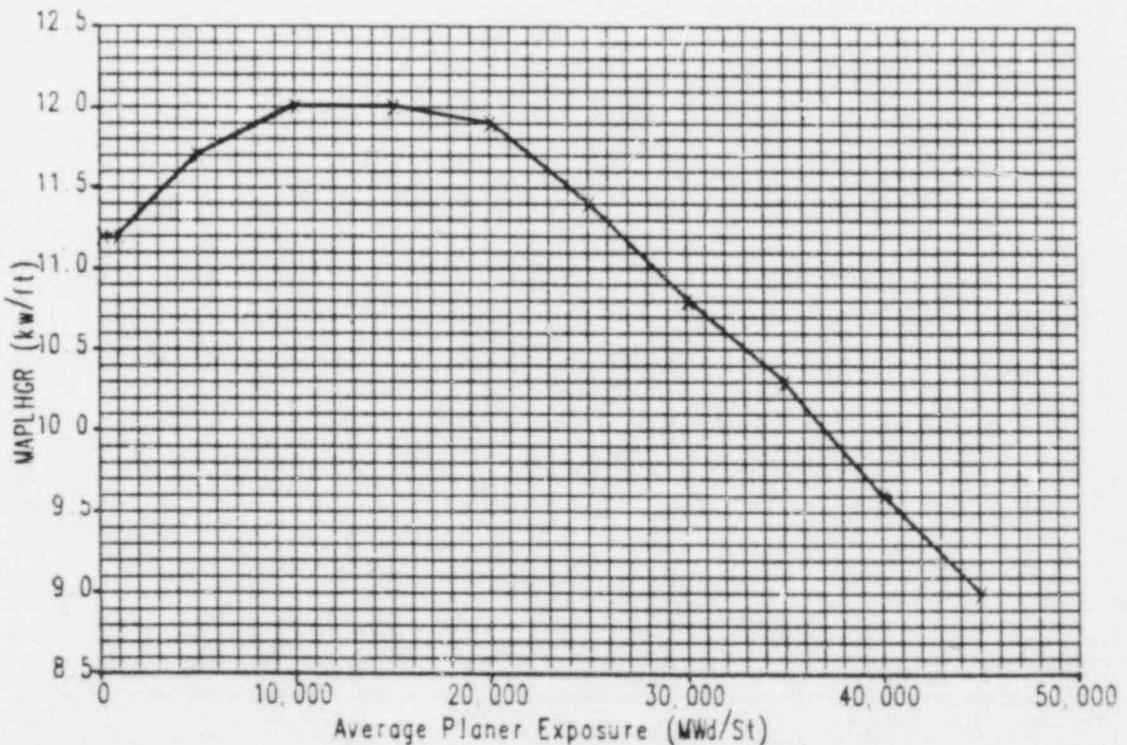
MAPLHGR Vs. Average Planer Exposure
Fuel Types P8DRB265H/BP8DRB265H



MAPLHGR Vs. Average Planer Exposure
Fuel Type BP8DRB282



MAPLHGR Vs. Average Planer Exposure
Fuel Type BP8DRB283H



QUAD CITIES
DPR-29

5.0 DESIGN FEATURES

5.1 Site

The Quad Cities Station, which consists of a tract of land of approximately 404 acres, is located about 3 miles north of Cordova, Illinois, Rock Island County, Illinois. The tract is situated in portions of Sections 7, 8, 17, and 18 of Township 20 North, Range 2 East.

5.2 Reactor

- A. The core shall consist of not more than 724 fuel assemblies.
- B. The reactor core shall contain 177 cruciform-shaped control rods. The control material shall be boron carbide powder (B_4C) compacted to approximately 70% of theoretical density or hafnium metal.

5.3 Reactor Vessel

The reactor vessel shall be as described in Table 4.1.1 of the SAR. The applicable design codes shall be as described in Table 4.1.1 of the SAR.

5.4 Containment

- A. The principal design parameters and applicable design codes for the primary containment shall be as given in Table 5.2.1 of the SAR.
- B. The secondary containment shall be as described in Section 5.3.2 of the SAR, and the applicable codes shall be as described in Section 12.1.1.3 of the SAR.
- C. Penetrations to the primary containment and piping passing through such penetrations shall be designed in accordance with standards set forth in Section 5.2.2 of the SAR.

5.5 Fuel Storage

- A. The new fuel storage facility shall be such that the K_{eff} dry is less than 0.90 and flooded is less than 0.95.
- B. The K_{eff} of the spent fuel storage pool shall be less than or equal to 0.95.

5.6 Seismic Design

The reactor building and all contained engineered safeguards are designed for the maximum credible earthquake ground motion with an acceleration of 24% of gravity. Dynamic analysis was used to determine the earthquake acceleration application to the various elevations in the reactor building.