



UNITED STATES
 NUCLEAR REGULATORY COMMISSION
 REGION II -
 101 MARIETTA STREET, N.W.
 ATLANTA, GEORGIA 30323

Report No.: 50-425/88-23

Licensee: Georgia Power Company
 P. O. Box 4545
 Atlanta, GA 30302

Docket No.: 50-425

Construction Permit No.: CPPR-109

Facility Name: Vogtle 2

Inspection Conducted: April 1 - May 5, 1988

Inspectors:	<u><i>J. Schepens</i></u>	<u>5/17/88</u>
	J. Schepens, Senior Resident Inspector	Date Signed
	<u><i>C. W. Burger</i></u>	<u>5/17/88</u>
	C. W. Burger, Resident Inspector	Date Signed
Approved by:	<u><i>M. V. Sinkule</i></u>	<u>5/17/88</u>
	M. V. Sinkule, Section Chief	Date Signed
	Division of Reactor Projects	

SUMMARY

Scope: This routine, unannounced inspection entailed resident inspection in the following areas: fire protection, reactor vessel and internals, piping systems and supports, safety related components, auxiliary systems, electrical equipment, raceways and instrumentation, preoperational testing, quality programs and administrative controls affecting quality, and allegations.

Results: No violations or deviations were identified.

REPORT DETAILS

1. Persons Contacted

Licensee Employees

- *P. D. Rice, Vice President, Vogtle Project Director
- *R. H. Pinson, Vice President, Project Construction
- G. Bockhold, General Manager
- T. V. Greene, Plant Support Manager
- C. W. Hayes, Vogtle Quality Assurance Manager
- *E. D. Groover, Quality Assurance Site Manager - Construction
- *D. M. Fiquett, Project Construction Manager
- *A. B. Gallant, Project Compliance Coordinator
- C. L. Coursey, Maintenance Superintendent (Startup)
- *H. M. Handfinger, Assistant Plant Support Manager
- A. W. Harrelson, Construction Electrical Manager
- C. W. Rau, Construction Mechanical Manager
- L. N. Brooks, Construction Civil Manager
- *I. D. Innes, Civil Project Section Supervisor
- L. B. Glenn, QC Manager
- *R. W. McManus, Readiness Review Manager
- J. E. Sanders, Assistant Project Manager
- W. C. Ramsey, Project Engineering Manager
- L. D. Harless, Manager, Quality Concerns

Other licensee employees contacted included craftsmen, technicians, supervision, engineers, inspectors, and office personnel.

Other Organizations

- C. Marcum, Westinghouse Project Manager
- D. L. Kinnsch, Project Engineering - Bechtel
- *T. E. Richardson, Project Engineering Manager - Bechtel
- A. J. Ayob, V&AMU Supervisor - Westinghouse
- B. Edwards, Site Manager - Pullman Power Products
- J. Miller, Quality Assurance Manager - Pullman Power Products
- *D. D. Smith, Construction Engineer - Oglethorpe Power Company
- *J. P. Hawley, Project Engineer - Bechtel
- *F. C. Ling, Engineering Group Leader - Bechtel
- *S. K. Owen, Project Engineer - Southern Company Services

*Attended Exit Interview

2. Exit Interviews - (30703C)

The inspection scope and findings were summarized on May 5, 1988, with those persons indicated in paragraph 1 above. The inspector described the areas inspected and discussed in detail the inspection results. No

dissenting comments were received from the licensee. The licensee did not identify as proprietary any of the materials provided to or reviewed by the inspector during this inspection. Region based NRC exit interviews were attended during the inspection period by a resident inspector.

3. Licensee Action on Previous Enforcement Matters - (92702)

Not inspected.

4. Allegations

(Closed) RII-86-A-302, Heating Ventilation And Air-Conditioning (HVAC) Concerns.

The inspector conducted a detailed review of the licensee response contained in GPC Letter No. GN-1438 dated, March 24, 1988, which addressed concerns relative to the engineering disposition on non conformance reports (NCR's) 225 and 226. Furthermore, a review was conducted and discussions were held with appropriate licensee personnel relative to the investigative material contained in quality concern file no. 88V0049. These documents consisted of NCR's 225 and 226. Based on this review, the inspector has concluded that the engineering dispositions contained in the subject NCR's were technically correct and addressed all of the discrepancies identified by the NCR's. Therefore, the concerns raised by the allegor pertained mainly with the improper identification of tag numbers for several fire dampers by the inspector originating the NCR's. The engineer's disposition did not translate these same errors nor was there an explanation in the NCR to address them. Therefore, the licensee has issued the appropriate paperwork as an attachment to the NCR's so as to explain the misidentification of tag numbers for the fire dampers in question. The inspector considers the licensee's action relative to this matter acceptable, therefore this item is considered to be closed since the above identified concern of fire damper misidentification had no safety impact on the plant.

5. General Construction Inspection - (92706)

Periodic random surveillance inspections were made throughout this reporting period in the form of general type inspections in different areas of the facility. The areas were selected on the basis of the scheduled activities and were varied to provide wide coverage. Observations were made of activities in progress to note defective items or items of noncompliance with the required codes and regulatory requirements. On these inspections, particular note was made of the presence of quality control inspectors, supervisors, and quality control evidence in the form of available process sheets, drawings, material identification, material protection, performance of tests, and housekeeping. Interviews were conducted with craft personnel, supervisors, coordinators, quality control inspectors, and others as they

were available in the work areas. The inspector reviewed numerous construction deviation reports to determine if requirements were met in the areas of documentation, action to resolve, justification, and approval signatures in accordance with GPC Field Procedure No. GD-T-01.

No violations or deviations were identified.

6. Fire Prevention/Protection and Housekeeping Measures - (42051C)

The inspector observed fire prevention/protection measures throughout the inspection period. Welders were using welding permits with fire watches and extinguishers. Fire fighting equipment was in its designated areas throughout the plant.

The inspector reviewed and examined portions of procedures pertaining to the fire prevention/protection measures and housekeeping measures to determine whether they comply with applicable codes, standards, NRC Regulatory Guides and licensee commitments.

The inspector observed fire prevention/protection measures in work areas containing safety related equipment during the inspection period to verify the following:

- Combustible waste material and rubbish was removed from the work areas as rapidly as practicable to avoid unnecessary accumulation of combustibles.
- Flammable liquids were stored in appropriate containers and in designated areas throughout the plant.
- Cutting and welding operations in progress have been authorized by an appropriate permit, combustibles have been moved away or safely covered, and a fire watch and extinguisher was posted as required.
- Fire protection/suppression equipment was provided and controlled in accordance with applicable requirements.

No violations or deviations were identified.

7. Reactor Coolant Pressure Boundary and Safety Related Piping - (49053C)
(49063C)

System Walkdown Inspection

Selected portions of the Reactor Coolant System (RCS) were walked down. The constructed piping system was compared to the piping and instrumentation diagrams. Examples of items checked were piping configuration, pipe class, weld location, and appearance type valve location and orientation, labeling, piping support location, vent and drain connections, instrumentation connections, and engagement of threaded fasteners.

The inspectors conducted the walkdown inspection with the piping and instrumentation diagram to verify that the system was constructed as shown on the current, approved revision and in accordance with the piping and instrumentation construction specification X4AZ01. The piping runs inspected were designed and constructed to ASME B & PVC, Section III Class I and II and seismic category 1 requirements. The following drawings were used in the walkdown.

<u>P&ID/Rev.</u>	<u>Title</u>
2X4DB111,R-12	Reactor Coolant System
2X4DB112,R-17	Reactor Coolant System

No violations or deviations were identified.

8. Reactor Coolant Pressure Boundary and Safety Related Piping Welding - (55073C) (55083C)

Periodic inspections were conducted during daily plant surveillances on safety-related pipe welding at various stages of weld completion. The purpose of the inspection was to determine whether the requirements of applicable specifications, codes, standards, work performance procedures and QC procedures are being met as follows:

- Work was conducted in accordance with a process sheet which identifies the weld and its location by system, references procedures or instructions, and provides for production and QC signoffs.
- Welding procedures, detailed drawings and instructions, were readily available in the immediate work area and technically adequate for the welds being made.
- Welding procedure specification (WPS) were in accordance with the applicable Code requirements and that a Procedure Qualification Record (PQR) is referenced and exists for the type of weld being made.
- Base metals, welding filler materials, fluxes, gases, and insert materials were of the specified type and grade, have been properly inspected, tested and were traceable to test reports or certifications.
- Purge and/or shielding gas flow and composition were as specified in the welding procedure specification and that protection was provided to shield the welding operation from adverse environmental conditions.
- Weld joint geometry including pipe wall thickness was specified and that surfaces to be welded have been prepared, cleaned and inspected in accordance with applicable procedures or instructions.

- A sufficient number of adequately qualified QA and QC inspection personnel were present at the work site, commensurate with the work in progress.
- The weld area cleanliness was maintained and that pipe alignment and fit-up tolerances were within specified limits.
- Weld filler material being used was in accordance with welding specifications, unused filler material was separated from other types of material and was stored properly and that weld rod stubs were properly removed from the work location.
- There were no evident signs of cracks, excessive heat input, sugaring, or excessive crown on welds.
- Welders were qualified to the applicable process and thickness, and that necessary controls and records were in place.

No violations or deviations were identified.

9. Reactor Vessel, Integrated Head Package, and Internals - (50053C)(50063C)

The inspection consisted of examinations of the reactor vessel installed in containment, the integrated head package and the lower and upper internals which were stored in their designated laydown areas. Inspections determined that proper storage protection practices were in place and that entry of foreign objects and debris was prevented.

During the inspection period, the reactor vessel upper and lower internals were installed in the reactor vessel and integrated head package was installed and tensioned to support of the reactor coolant system primary hydrostatic test. The inspector witnessed various portions of the assembly process which included final inspections of the vessel internals and integrated package prior to installation. The inspection consisted of but was not limited to the following:

- Visual examination of accessible welds to verify no sign of cracks.
- Visual examination of accessible locking devices to verify proper crimping and no damage.
- Visual examination of O-ring seating surfaces to verify clearness and no damage.
- Visual examination of lower & upper internals to verify clearness prior to installation.

No violations or deviations were identified.

10. Safety Related Components - (50073C)

The inspection consisted of plant tours to observe storage, handling, protection, installation, and preventive maintenance after installation of safety-related components to determine that work is being performed in accordance with applicable codes, NRC Regulatory Guides, and licensee commitments.

During the inspection the below listed areas were inspected at various times during the inspection period to verify the following as applicable:

- Storage, environment, and protection of components were in accordance with manufacturer's instructions and/or established procedures.
- Implementation of special storage and maintenance requirements such as: rotation of motors, pumps, lubrication, insulation testing (electrical), cleanliness, etc.
- Performance of licensee/contractor surveillance activities and documentation thereof was being accomplished.
- Installation requirements were met such as: proper location, placement, orientation, alignment, mounting (torquing of bolts and expansion anchors), flow direction, tolerances, and expansion clearance.
- Appropriate stamps, tags, markings, etc. were in use to prevent oversight of required inspections, completion of tests, acceptance, and the prevention of inadvertent operation.

Safety-Related piping, valves, pumps, heat exchangers, and instrumentation were inspected in the following areas on a random sampling basis throughout the inspection period:

- Residual Heat Removal Pump Rooms
- Diesel Generator Building
- Auxiliary Feedwater Pumphouse
- Containment Spray Pump Rooms
- Pressurizer Rooms
- Main Coolant Pump Areas
- Steam Generator Areas
- Safety Injection Pump Rooms
- RHR and CS Containment Penetration Encapsulation Vessel Rooms
- Component Cooling Water (CCW) Heat Exchangers, Surge Tanks & Pump Rooms
- Cable Spreading Rooms
- Accumulator Tank Areas
- Chemical and Volume Control System (CVCS) Letdown Heat Exchanger Pump Room
- Nuclear Grade Piping, Valves & Fittings Storage Areas

- Pressurizer Relief Tank Area
- CVCS Centrifugal Charging Pumps & Positive Displacement Pump Rooms
- NSCW Tower Pump Rooms and Pipe Tunnels

An inspection was conducted of the Train "A" diesel generator fuel oil day tank which consisted of a review of the design, fabrication, and installation process. FSAR Section 9.5.3.2.1.3 specifies the storage capacity, and the design configuration of the DG fuel oil day tank within the diesel generator building. The following documents were reviewed by the inspector to verify the proper implementation of the design, fabrication, and installation of the train "A" diesel generator fuel oil day tank.

<u>Document No./Rev.</u>	<u>Title</u>
DC 2403, R-3, DMCN-5	DG Fuel Oil Day Tank Design Basis
X4AH04, R-13	Construction Specification For DG Fuel Oil Day Tank

<u>Document No./Rev.</u>	<u>Title</u>
2X4DL58D01, R-5	DG Building Area Piping
ZX4DL58D05, R-4	DG Building Area Piping
X4C2403S00F1, R-1	Failure Modes And Effects Analysis Stdby DG Engine Fuel Oil Storage And Transfer System
2X4AH04-21-14	DG Fuel Oil Day Tank Fabrication Drawing
2-2403-T4-003	Field Process Sheet For Installation Of The Train "A" DG Fuel Oil Day Tank

No violations or deviations were identified.

11. Safety Related Pipe Support and Restraint Systems - (50090C)

Periodic random inspections were conducted during the inspection period to observe construction activities during installation of safety-related pipe supports to determine that the following work was performed in accordance with applicable codes, NRC Regulatory Guides, and licensee commitments:

- Spring hangers were provided with indicators to show the approximate "hot" or "cold" position, as appropriate.
- No deformation or forced bending was evident.
- Where pipe clamps are used to support vertical lines, shear lugs were welded to the pipe (if required by Installation Drawings) to prevent slippage.

- Sliding or rolling supports were provided with material and/or lubricants suitable for the environment and compatible with sliding contact surfaces.
- Supports are located and installed as specified.
- The surface of welds meet applicable code requirements and are free from unacceptable grooves, abrupt ridges, valleys, undercuts, cracks, discontinuities, or other indications which can be observed on the welded surface.

No violations or deviations were identified.

12. Electrical and Instrumentation Components and Systems - (51053C)(52153C)

Periodic inspections were conducted during the inspection period to observe safety-related electrical equipment in order to verify that the storage, installation, and preventive maintenance was accomplished in accordance with applicable codes, NRC Regulatory Guides, and licensee commitments.

During the inspection period inspections were performed on various pieces of electrical equipment during storage, installation, and cable terminating phase in order to verify the following as applicable:

- Location and alignment
- Type and size of anchor bolts
- Identification
- Segregation and identification of nonconforming items
- Location, separation and redundancy requirements
- Equipment space heating
- Cable identification
- Proper lugs used
- Condition of wire (not nicked, etc.), tightness of connection
- Bending radius not exceeded
- Cable entry to terminal point
- Separation

No violations or deviations were identified.

13. Electrical and Instrumentation Cables and Terminations - (51063C)(52063C)
(53053C)

a. Raceway/Cable Installation

The inspector reviewed and examined portions of the following procedures pertaining to raceway/cable installation to determine whether they comply with applicable codes, NRC Regulatory Guides and licensee commitments.

- ED-T-02, Raceway Installation
- ED-T-07, Cable Installation

Periodic inspections were conducted to observe construction activities of Safety Related Raceway/Cable Installation.

In reference to the raceway installation, the following areas were inspected to verify compliance with the applicable requirements:

- Identification
- Alignment
- Bushings (Conduit)
- Grounding
- Supports and Anchorages

In reference to the cable installation the following areas were inspected to verify compliance with the applicable requirements:

- Protection from adjacent construction activities (welding, etc.)
- Coiled cable ends properly secured
- Non-terminated cable ends taped
- Cable trays, junction boxes, etc., reasonably free of debris
- Conduit capped, if no cable installed
- Cable supported
- Bend radius not exceeded
- Separation

During the inspection period, seismic gap separation inspections were conducted for commodities attached to either the same or adjacent building structure in accordance with the criteria specified in table 5 of GPC field procedure ED-T-02 "Raceway Installation." The purpose of the inspection was to verify that the minimum acceptance clearance for seismic separation between commodities was being maintained during plant construction. The following areas were inspected:

Control Building - Levels A & B

<u>Room No.</u>	<u>Description</u>	<u>Seismic Gap Structures</u>
02	Elec. Pen. Rm.	Cntrl. To Cnmt. Bldg.
10	Elec. Pen. Rm.	Cntrl. To Cnmt. Bldg.
11	Elec. Pen. Rm.	Cntrl. To Cnmt. Bldg.
12	Elec. Pen. Rm.	Cntrl. To Cnmt. Bldg.
13	Train "B" Elec. Tunnel	Cntrl. To Aux. Bldg.
19	Elec. Pen. Rm.	Cntrl. To Cnmt. Bldg.

In addition, discussions were held with licensee personnel regarding the results of their preliminary seismic gap inspection walkdown findings of suspect areas such as fuel handling building to auxiliary building, auxiliary building to containment building (penetration

rooms), and containment building to liner plate. These preliminary inspections were conducted ahead of the inspections which will be conducted as part of the finalization program based on lessons learned from unit 1. The licensee has initiated the appropriate paperwork to evaluate and / or correct the identified deficiencies.

b. Cable Terminations

The inspector reviewed and examined portions of the following procedures pertaining to cable termination to determine whether they comply with applicable codes, NRC Regulatory Guides and licensee commitments.

- ED-T-08, Cable Termination

In reference to cable terminations the following areas were inspected to verify compliance with the applicable requirements.

- Cable identification
- Proper lugs used
- Condition of wire (not nicked, etc.)
- Tightness of connection
- Bending radius not exceeded
- Cable entry to terminal point
- Separation

An inspection was conducted of completed Raychem head shrink tubing installations for cable terminations inside containment. The purpose of the inspection was to verify that no burn damage existed as a result of the application of excessive heat. The following cables were visually inspected:

Containment Building - Level B

<u>Junction No.</u>	<u>Cable No.</u>	<u>Equipment Servicing</u>
2BRJB4851	2BR10951SB	2HV-10951
2ARJB4852	2AR10952SB	2HV-10952
2BRJB4853	2BR10953SB	2HV-10953
2BRJB4869	2BR88241WA 2BR88241SC 2BR88241SD 2BR88241SE 2BR88241SF	2HV-8824

2BJJB3711	2BRJ8209SA	2HV-8209
	2BRJ8209SB	2HV-8211
	2BRJ8209WB	
	2BRJ8211SA	
	2BRJ8211SB	
	2BRJ8211TC	
	2BRJ8211WB	

No violations or deviations were identified.

14. Reactor Coolant System Hydrostatic Test - (70362)

This inspection consisted of a review of the applicable procedures listed below to ascertain the overall scope, technical adequacy, and that the test is consistent with regulatory requirements, guidance and licensee commitments. Discussions were held with the principal test directors to ascertain their knowledge of how the test is to be conducted.

The following procedures reviewed:

<u>Procedure No.</u>	<u>Title</u>
2-300-03, Rev 1	RCS Primary Hydrostatic Test
2-1201-010	PPP Hydro Package
13001-2, Rev 0	RCS Filling and Venting
13003-2, Rev 0	Reactor Coolant Pump Operation

Regulatory requirements, guidance and licensee commitments as applicable were:

<u>Documents</u>	<u>Title</u>
FSAR Table 14.2.8.1.9	RCS Hydrostatic Preoperational Test
Regulatory Guide 1.68, Rev 2	Initial Test Programs for Water-Cooled Plants
10 CFR 50.2 (V)	RCS Pressure Boundary
Draft Tech Spec 3/4.4.9	RCS Pressu: Temperature Limits
ASME, Section III, Div 1, NB	ASME Boiler and Pressure Vessel Code

The inspector's review concluded that the test procedure contained the following attributes and/or requirements:

- Definition of the test boundary
- Venting the reactor coolant system during the filling operation
- Water quality chemistry requirements were specified
- Temperature ranges and heatup rates and pressurization rates were specified
- Minimum and maximum test pressures were properly calculated

- The hydrostatic test pressure will be maintained for a minimum of ten minutes before initiation of the examination for leakage
- Examinations for leakage will be conducted at design pressure with an appropriate range
- Appropriate inspection points, N-stamp documentation and QC verification of test results

The inspection also included a control room board walkdown of the instrumentation which will be utilized during the execution of the procedures.

No violations or deviations were identified.

15. Preoperational Test Program Implementation/Verification (70302)(71302)

The inspector reviewed the present implementation of the preoperational test program. Test program attributes inspected included review of administrative requirements, document control, documentation of major test events and deviations to procedures, operating practices, instrumentation calibrations, and correction of problems revealed by testing.

Periodic inspections were conducted of Control Room Operations to assess plant condition and conduct of shift personnel. The inspector observed that Control Room operations were being conducted in an orderly and professional manner. Shift personnel were knowledgeable of plant conditions, i.e., ongoing testing, systems/equipment in or out of service, and alarm/annunciator status. In addition, the inspector observed shift turnovers on various occasions to verify the continuity of plant testing, operational problems and other pertinent plant information during the turnovers. Control Room logs were reviewed and various entries were discussed with operations personnel.

Periodic facility tours were made to assess equipment and plant conditions, maintenance and preoperational activities in progress. Schedules for program completion and progress reports were routinely monitored. Discussions were held with responsible personnel, as they were available, to determine their knowledge of the preoperational program. The Inspector reviewed numerous operation deviation reports to determine if requirements were met in the areas of documentation, action to resolve, justification, corrective action and approvals. Specific inspections conducted are listed below:

a. Preoperational Tests

(1) Test Procedure Review (70300)

The inspector reviewed the following listed preoperational test procedures. Each test was reviewed for administrative format and technical adequacy. The procedures were compared with licensee commitments from the applicable FSAR Chapters,

Regulatory Guide 1.68 and the Safety Evaluation Report (NUREG-1137). This included verifying that pertinent prerequisites were identified, initial test conditions and system status were specified, acceptance criteria were specified and management approval indicated:

<u>Procedure No.</u>	<u>NRC Insp. No.</u>	<u>Test Title</u>
2-3BB-05	70347	Pressurizer Pressure & Level Control System Preoperation Test Procedure
2-3BB-08	70308	Reactor Vessel And Internals Preoperational Test Procedure

(2) Test Witnessing (70312)

The inspector witnessed selected portions of the following preoperational test procedures as they were conducted. The inspection included attendance at briefings held by the test supervisor to observe the coordination and general knowledge of the procedure with the test participants. Overall crew performance was evaluated during testing. A preliminary review of the test results was compared to the inspector's own observations. Problems encountered during performance of the test were verified to be adequately documented, evaluated and dispositioned on a selected basis.

<u>Procedure No.</u>	<u>NRC Insp. No.</u>	<u>Test Title</u>	<u>Activity Observed</u>
2-3BK-01,R-0	70443	Containment Spray (CS)	CS Pumps And System Operation Evaluation - Section 6.4
2-3BK-01,R-0	70443	Containment Spray	Suction Flow Demonstration For Simultaneous Operation Of SI, CC, RHR, And CS Pumps
2-3EC-01,R-0	71312	Spent Fuel Pool Cleaning	Spent Fuel Pool Pump Breaker Control Logic Checkout Of Pump Nos. 2 & 5 - Section 6.0

2-3BK-01,R-0	70443	Containment Spray	CS Additive Subsystem And Educators Performance Test
2-3BN-01, R-0	71312	Refueling Water Storage Tank	Sludge Mixing Pump And Electric Circulation Heater Controls And Performance Test

b. Construction Acceptance Testing (CAT)

The inspector witnessed portions of the following CAT's to verify implementation per Startup Manual Procedure No. SUM-12A:

<u>Cat No.</u>	<u>Description</u>
87-3121	Train "B" Auxiliary Feed Water Pump Initial Operation Per CAT-M-01

During the inspection period the inspector observed the licensee's performance during the vendor break-in runs of the train "B" diesel generator per Startup Operating Instruction (SOI), SOI-2KJ-01 "Standby Diesel Generator 2B Initial Startup" which detailed the necessary instructions for the performance of a break-in run and vendor check out during initial operation. The testing consisted of operating the diesel generator at various loads for specified time periods up to and including 110% load (7700KW) and the performance of various internal engine component inspections to ensure operation. Also, the inspectors observed the train "A" and "B" diesel generator 7 day run. During this phase it was necessary for the licensee to shutdown the diesel generator periodically to make minor repairs and / or adjustments. This phase of operation was conducted to acquire engine hours of operation prior to performing additional inspections of various engine components. The inspector observed the replacement of the train "B" diesel generator turbocharger bearings.

The inspector observed the operation of the unit 2 control room engineered safety features fans in the recirculation mode without pressurization in accordance with standing order no. C-88-03 and SOI Nos. 47 and 49. The inspector verified that communication was established with head phones between unit 1 and 2.

No violations or deviations were identified.

16. Management Meetings - (30702)

This activity involves inspector participation and preparation in support of the following meetings which presented site readiness.

The resident inspector attended a presentation given by the licensee to Region II management at Vogtle Generating Plant. In attendance from the regional office were Mr. A. R. Herdt, Engineering Branch Chief - Division of Reactor Safety and Mr. M. D. Hunt, Electrical Inspector - DRS. The licensee presented their plan and current status regarding fire hazard analysis, finalization program, design/installation and nuclear operations interface. This presentation was useful in that it provided the staff with the necessary information for planning the upcoming Appendix "R" inspection in support of licensing.

17. Meeting With Local Officials - (94600)

On May 3, a meeting was held to familiarize local officials of the City of Waynesboro and Burke County Board of Commissioners with the NRC. The meeting presented the mission of the NRC, introduction of key NRC personnel, discussion of lines of communication available to local officials, and discussion of the facility status. The NRC presented the scope of the inspection and emergency preparedness programs.

The meeting was open for questions during the presentation. NRC persons involved with the presentation were as follows:

- M. V. Sinkule, Chief, Reactor Projects Section 3B, Division of Reactor Projects
- J. F. Rogge, Senior Resident Inspector, Operations
- R. J. Schepens, Senior Resident Inspector, Construction
- C. W. Burger, Resident Inspector