October 14, 1998

LICENSEE:

Florida Power Corporation

FACILITY:

Crystal River, Unit 3

SUBJECT:

SUMMARY OF MEETING ON SEPTEMBER 21, 1998, CONCERNING CRYSTAL RIVER UNIT 3 EMERGENCY FEEDWATER SYSTEM AND

HIGH PRESSURE INJECTION SYSTEM MODIFICATIONS

On September 21, 1998, a meeting was held between staff of the U.S. Nuclear Regulatory Commission (NRC) and representatives of Florida Power Corporation (FPC). The purpose of the meeting was to discuss FPC plans to modify the emergency feedwater system and the high pressure injection system to resolve previously identified design issues related to diesel generator loading and operator burden as a result of required manual actions for a small break loss of coolant accident. The presentation included tentative schedules for the submittal of necessary license amendment requests and the date by which approval of these amendments would be needed. Enclosure 1 is an attendance list for the meeting. Enclosure 2 includes copies of the meeting handouts.

The NRC staff indicated that the information presented by FPC at the meeting, as described in the enclosed handouts, was very useful in understanding the scope, schedule and potential impact on NRC resources of the projects. The NRC representatives indicated that is would be useful to include a discussion of risk reduction and risk insights in the license submittals. They also stated that a description of post-modification testing should be included in the submittals. The NRC staff stressed the need for timely submittal of the license amendment requests to provide adequate time for review and processing. The meeting was adjourned after a brief question-and-answer session.

151

Leonard A. Wiens, Senior Project Manager Project Directorate 1:3 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures: 1. Attendance List

2. Handouts

cc w/enclosures: See next page

DISTRIBUTION:

Hard Copy

E-MAIL

Docket File PUBLIC Crystal r/f

SCollins/FMiraglia BBoger

TMartin MTschiltz LPlisco, RII

JTatum WLeFave JKnox

L.V.jens ACRS

FHebdon BClayton

JZwolinski

EWeiss

SSaba

OGC

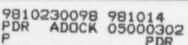
DOCUMENT NAME: G:\CRYSTAL\MEETGSUM.EFW

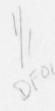
NAC FILE CENTER COPY

To receive a copy of this document, indicate in the box: "C" = Copy without attachment/enclosure "E" = Copy with attachment/enclosure "N" = No copy

OFFICE	P011-3/PM C	PD11-3/LA	PD11-3/D) C	
NAME	LWiens:cw for	BClayton BOC	FHebdon 💮	
DATE	10//4/98	10/ /4/98	10/14/98	10/ /98

OFFICIAL RECORD COPY





October 14, 1998

LICENSEE:

Florida Power Corporation

FACILITY:

Crystal River, Unit 3

SUBJECT:

SUMMARY OF MEETING ON SEPTEMBER 21, 1998, CONCERNING CRYSTAL RIVER UNIT 3 EMERGENCY FEEDWATER SYSTEM AND

HIGH PRESSURE INJECTION SYSTEM MODIFICATIONS

On September 21, 1998, a meeting was held between staff of the U.S. Nuclear Regulatory Commission (NRC) and representatives of Florida Power Corporation (FPC). The purpose of the meeting was to discuss FPC plans to modify the emergency feedwater system and the high pressure injection system to resolve previously identified design issues related to diesel generator loading and operator burden as a result of required manual actions for a small break loss of coolant accident. The presentation included tentative schedules for the submittal of necessary license amendment requests and the date by which approval of these amendments would be needed. Enclosure 1 is an attendance list for the meeting. Enclosure 2 includes copies of the meeting handouts.

The NRC staff indicated that the information presented by FPC at the meeting, as described in the enclosed handouts, was very useful in understanding the scope, schedule and potential impact on NRC resources of the projects. The NRC representatives indicated that is would be useful to include a discussion of risk reduction and risk insights in the license submittals. They also stated that a description of post-modification testing should be included in the submittals. The NRC staff stressed the need for timely submittal of the license amendment requests to provide adequate time for review and processing. The meeting was adjourned after a brief question-and-answer session.

151

Leonard A. Wiens, Senior Project Manager Project Directorate II-3 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures: 1. Attendance List

2. Handouts

cc w/enclosures: See next page

DISTRIBUTION:

Hard Copy

E-MAIL

Docket File

SCollins/FMiraglia

TMartin

JTatum WI eFav

PUBLIC Crystal r/f

BBoger JZwolinski MTschiltz LPlisco, RII WLeFave JKnox

LWiens ACRS

FHebdon BClayton EWeiss SSaba JKnox

OGC

DOCUMENT NAME: G:\CRYSTAL\MEETGSUM.EFW

To receive a copy of this document, indicate in the box: "C" = Copy without attachment/enclosure "E" = Copy with attachment/enclosure "N" = No copy

OFFICE	PD11-3/PM	C P011-3/LA	PD11-3/0	Icl	
NAME	LWiens: CM ft V	BClayton 3/1	FHebdon 💜		
DATE	10/14/98	10/ /4/98	10/14/98		10/ /98



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 14, 1998

LICENSEE:

Florida Power Corporation

FACILITY:

Crystal River, Unit 3

SUBJECT:

SUMMARY OF MEETING ON SEPTEMBER 21, 1998, CONCERNING CRYSTAL RIVER UNIT 3 EMERGENCY FEEDWATER SYSTEM AND

HIGH PRESSURE INJECTION SYSTEM MODIFICATIONS

On September 21, 1998, a meeting was held between staff of the U.S. Nuclear Regulatory Commission (NRC) and representatives of Florida Power Corporation (FPC). The purpose of the meeting was to discuss FPC plans to modify the emergency feedwater system and the high pressure injection system to resolve previously identified design issues related to diesel generator loading and operator burden as a result of required manual actions for a small break loss of coolant accident. The presentation included tentative schedules for the submittal of necessary license amendment requests and the date by which approval of these amendments would be needed. Enclosure 1 is an attendance list for the meeting. Enclosure 2 includes copies of the meeting handouts.

The NRC staff indicated that the information presented by FPC at the meeting, as described in the enclosed handouts, was very useful in understanding the scope, schedule and potential impact on NRC resources of the projects. The NRC representatives indicated that is would be useful to include a discussion of risk reduction and risk insights in the license submittals. They also stated that a description of post-modification testing should be included in the submittals. The NRC staff stressed the need for timely submittal of the license amendment requests to provide adequate time for review and processing. The meeting was adjourned after a brief question-and-answer session.

Leonard A. Wiens, Senior Project Manager

Project Directorate II-3

Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures: 1. Attendance List

2. Handouts

cc w/enclosures: See next page

Florida Power Corporation

Mr. John Paul Cowan, Vice President
Nuclear Operations (SA2A)
ATTN: Manager Nuclear Licensing
Florida Power Corporation
Crystal River Energy Complex
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Mr. R. Alexander Glenn Corporate Counsel Florida Power Corporation MAC-A5A P.O. Box 14042 St. Petersburg, Florida 33733-4042

Mr. Charles G. Pardee, Director Nuclear Plant Operations (NA2C) Florida Power Corporation Crystal River Energy Complex 15760 W. Power Line Street Crystal River, Florida 34428-6708

Mr. Robert B. Borsum Framatome Technologies Inc. 1700 Rockville Pike, Suite 525 Rockville, Maryland 20852

Mr. William A. Passetti, Chief Department of Health Bureau of Radiation Control 2020 Capital Circle, SE, Bin #C21 Tallahassee, Florida 32399-1741

Attorney General Department of Legal Affairs The Capitol Tallahassee, Florida 32304

Mr. Joe Myers, Director Division of Emergency Preparedness Department of Community Affairs 2740 Centerview Drive Tallahassee, Florida 32399-2100

CRYSTAL RIVER UNIT NO. 3 GENERATING PLANT

Chairman
Board of County Commissioners
Citrus County
110 North Apopka Avenue
Inverness, Florida 34450-4245

Mr. Robert E. Grazio, Director Nuclear Regulatory Affairs (SA2A) Florida Power Corporation Crystal River Energy Complex 15760 W. Power Line Street Crystal River, Florida 34428-6708

Senior Resident Inspector Crystal River Unit 3 U.S. Nuclear Regulatory Commission 6745 N. Tallahassee Road Crystal River, Florida 34428

Mr. Gregory H. Halnon
Director, Quality Programs (SA2C)
Florida Power Corporation
Crystal River Energy Complex
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Regional Administrator, Region II U.S. Nuclear Regulatory Commission 61 Forsyth Street, SW., Suite 23T85 Atlanta, GA 30303-3415

Mr. Leonard D. Wert U.S. Nuclear Regulatory Commission 61 Forsyth Street, SW., Suite 23T85 Atlanta, GA 30303-3415

ATTENDEES

SEPTEMBER 21, 1998 MEETING

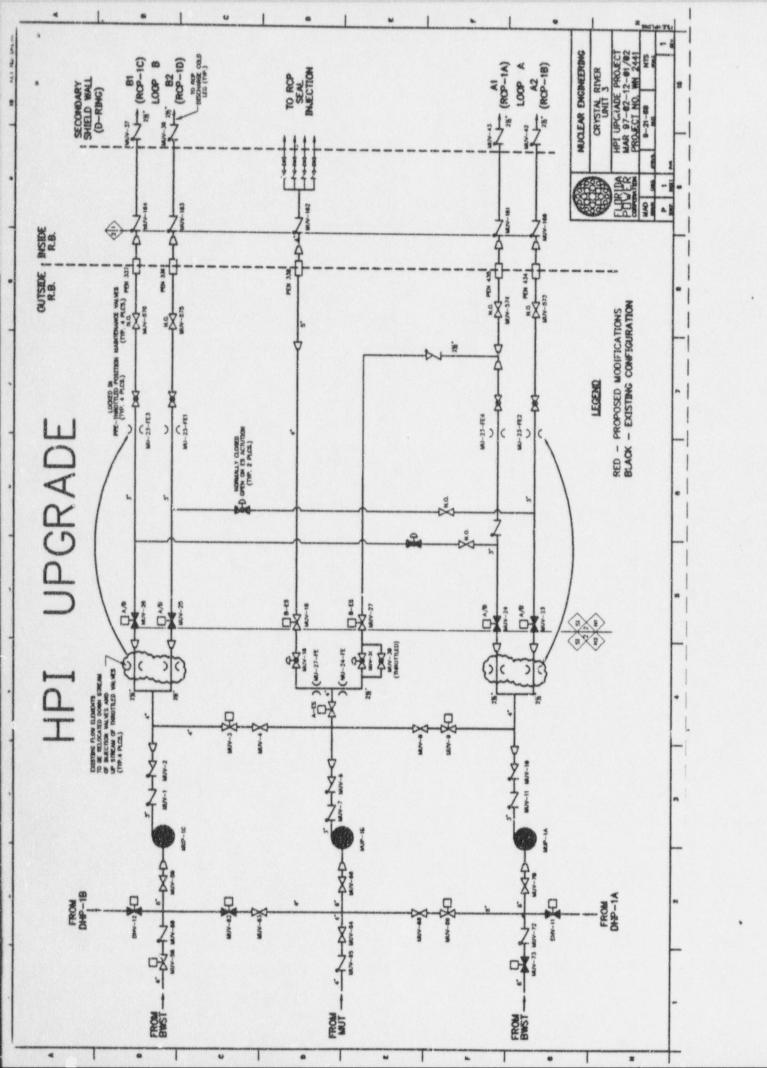
CRYSTAL RIVER

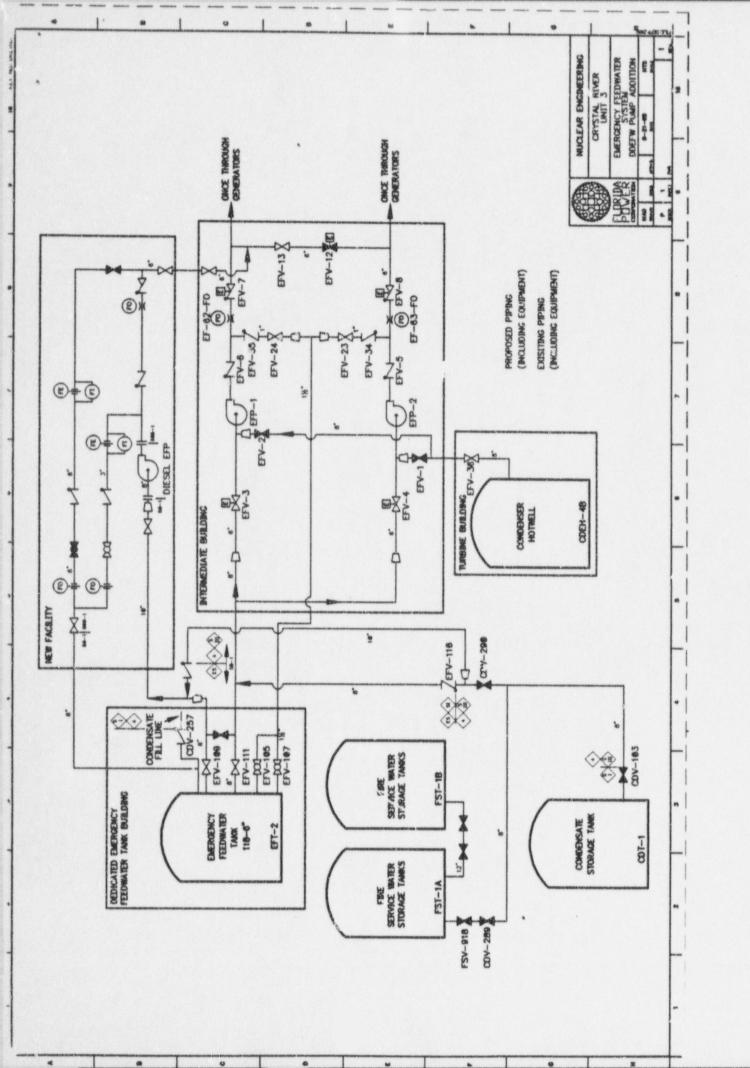
HIGH PRESSURE INJECTION

AND

EMERGENCY FEEDWATER

Name Affiliation Scot Greenlee FPC Craig Miller FPC Tim Stack FPC/Framatome Technologies Sherry Bernhoft FPC Cecil T. Gurganus FPC Paul V. Fleming FPC Eric Weiss NRC/SRXB S. N. Saba NRC/EELB Jim Tatum NRC/SPLB Bill LeFave NRC/SPLB John Knox NRC/EELB







Florida Power Corporation

Presentation to the Nuclear Regulatory Commission

Diesel Driven Emergency Feedwater Pump and HPI Upgrade Projects

September 21, 1998



Diesel Driven Emergency Feedwater Pump and HPI Upgrade Projects

Agenda

- > Introduction Scot Greenlee
- > Diesel Driven Emergency Feedwater Pump
 - » System Operation Paul Fleming
 - » Project Details Craig Miller
- > HPI Upgrade Project
 - » System Operation Paul Fleming
 - » Project Details Craig Miller
- > Conclusion Scot Greenlee
- Questions and Answers Open Format



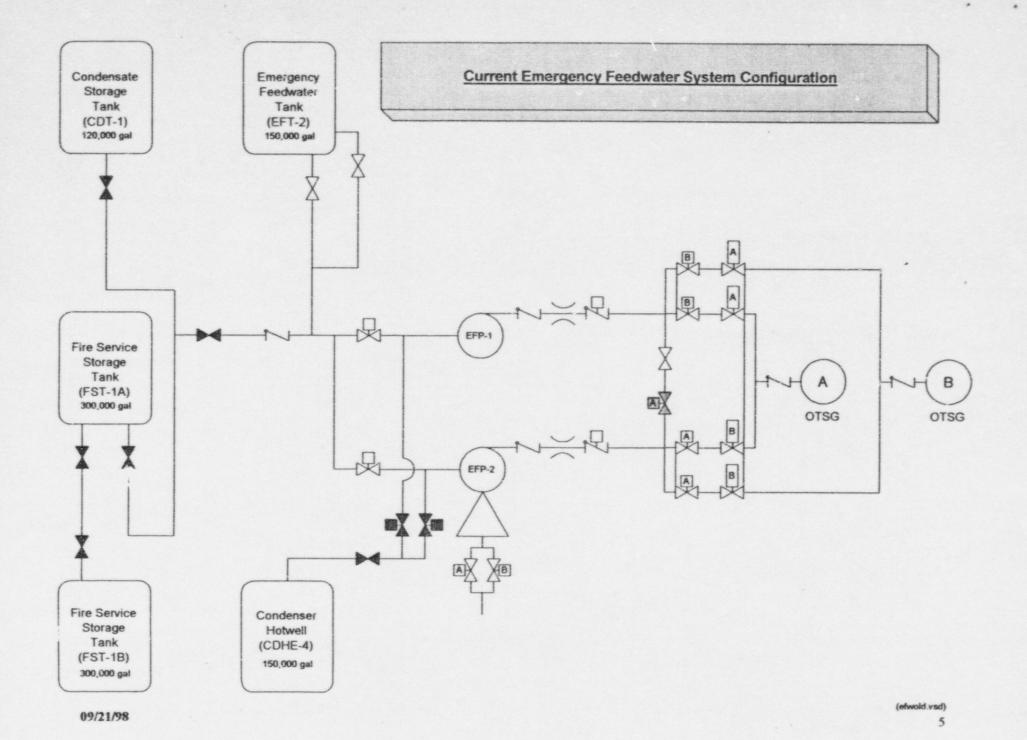
Diesel Driven Emergency Feedwater Pump and HPI Upgrade Projects

■ Introduction

- > Purpose
 - » Scope and Schedule for Projects
 - » NRC Resource Impacts
- Background
 - » Design Outage Issues
 - » FPC Commitments
- > Modification Overview
 - » EFW System Upgrade
 - » HPI System Upgrade



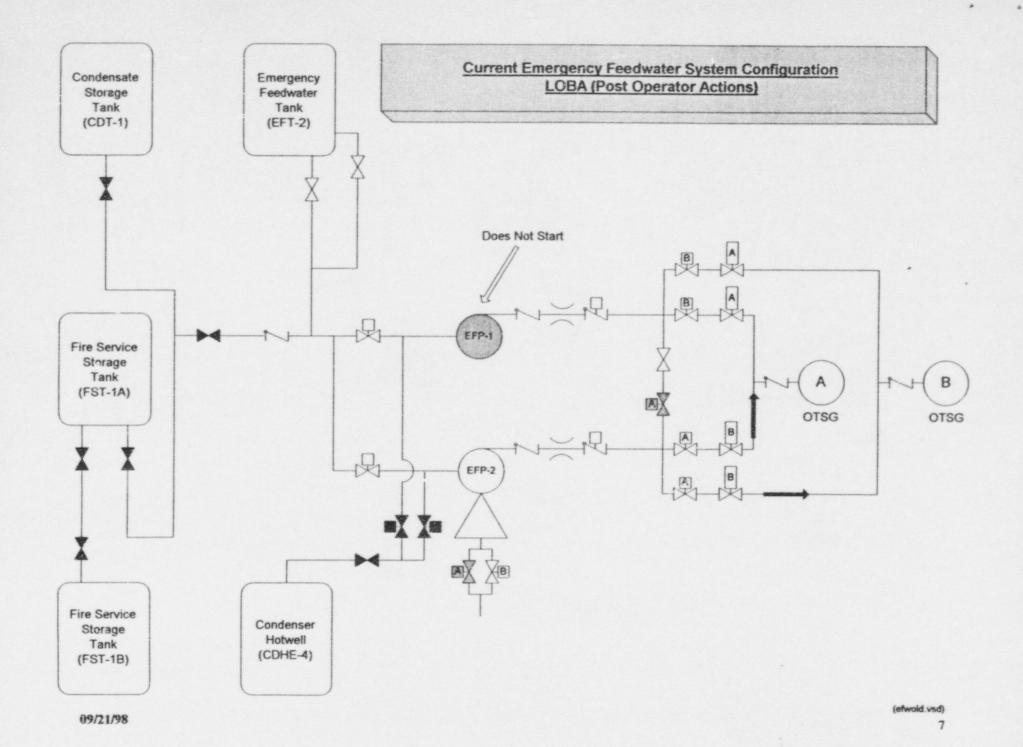
- EFW Configurations and Operation
 - License Amendment No. 163
 - > SBLOCA
 - » Spectrum Requiring Use of OTSG Cooling / Emergency Feedwater
 - Operator Actions Required to Mitigate Design Basis Accidents
 - » EDG Load Management
 - » EFW Configuration Management

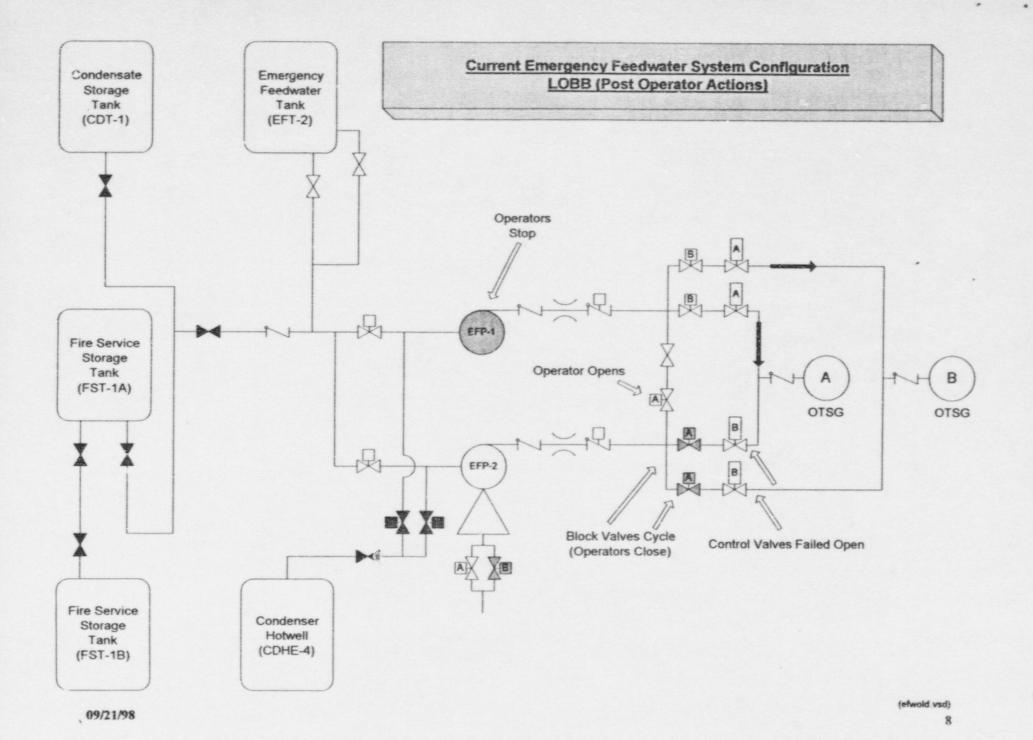


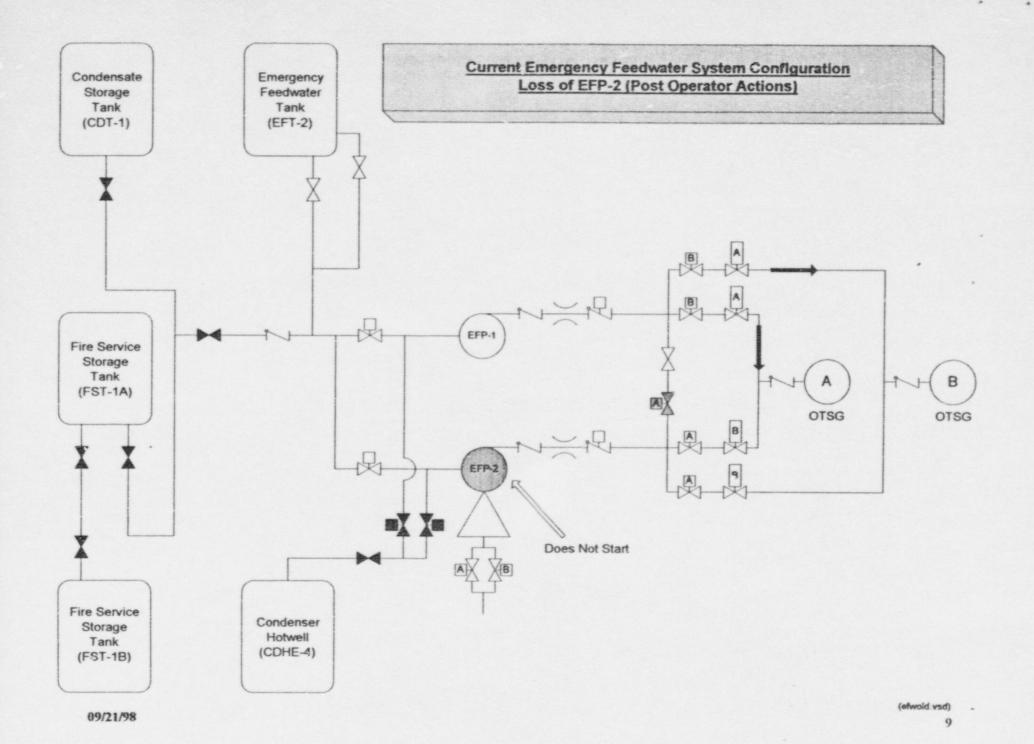


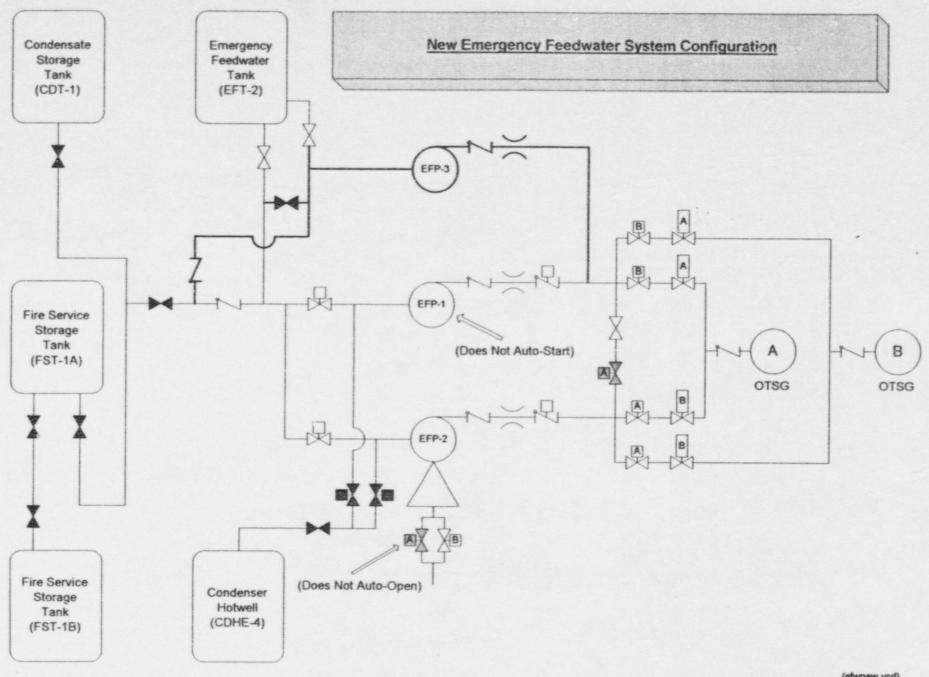
■ SBLOCA - LOOP Challenging Single Failures with Current Configuration

- > Loss of Battery "A" [LOBA]
 - » "A" Train ES Equipment Inoperable
 - » "B" Train ES Equipment Operable
 - » EFP-2 Starts (via ASV-5)
- Loss of Battery "B" [LOBB]
 - » "A" Train ES Equipment Operable
 - » "B" Train ES Equipment Inoperable
 - » EFP-1 Starts
 - » EFP-2 Starts (via ASV-204)
- > Loss of EFP-2
 - » Both Trains ES Equipment Operable
 - » EFP-1 Starts

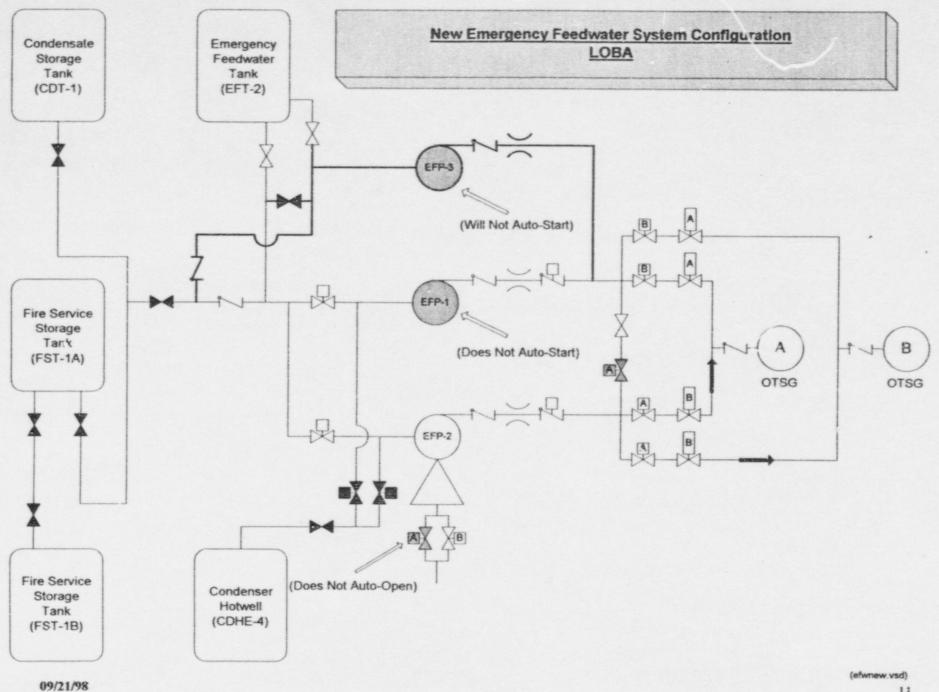


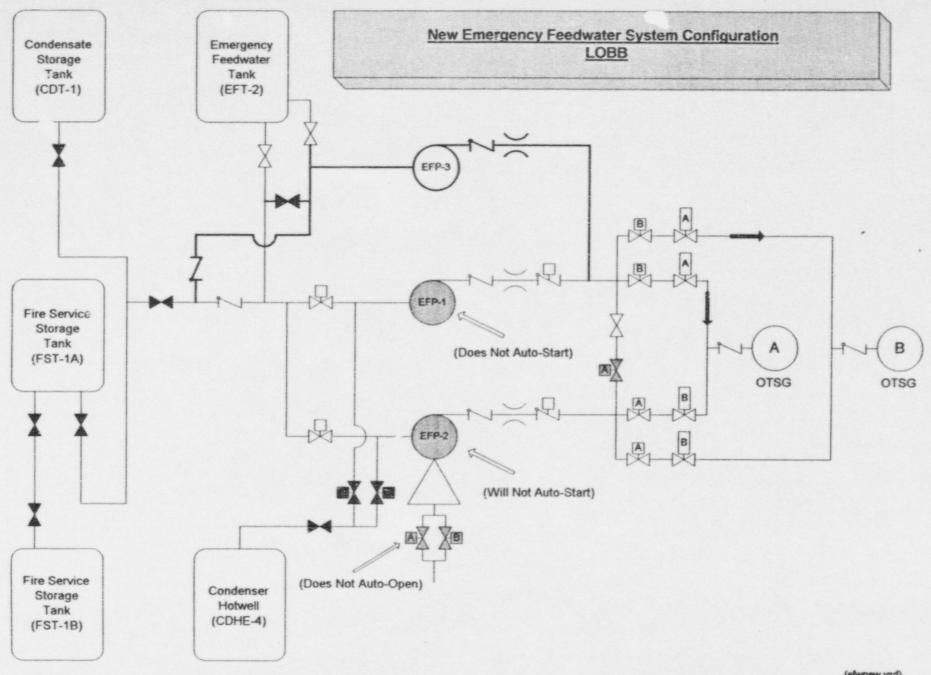




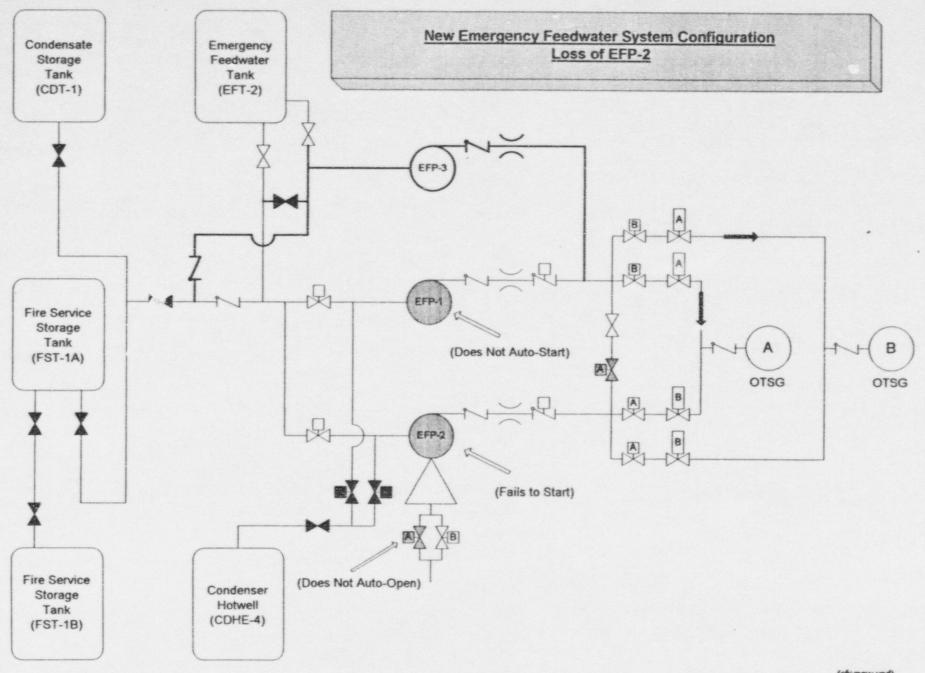


09/21/98





09/21/98



09/21/98



Design Alternatives

- > Detailed Study
 - » Replace or Upgrade Both EDGs
 - » Replace Single EDG
 - » Add Third EDG
 - » Add Diesel Driven EFP
- > Third Party Evaluation



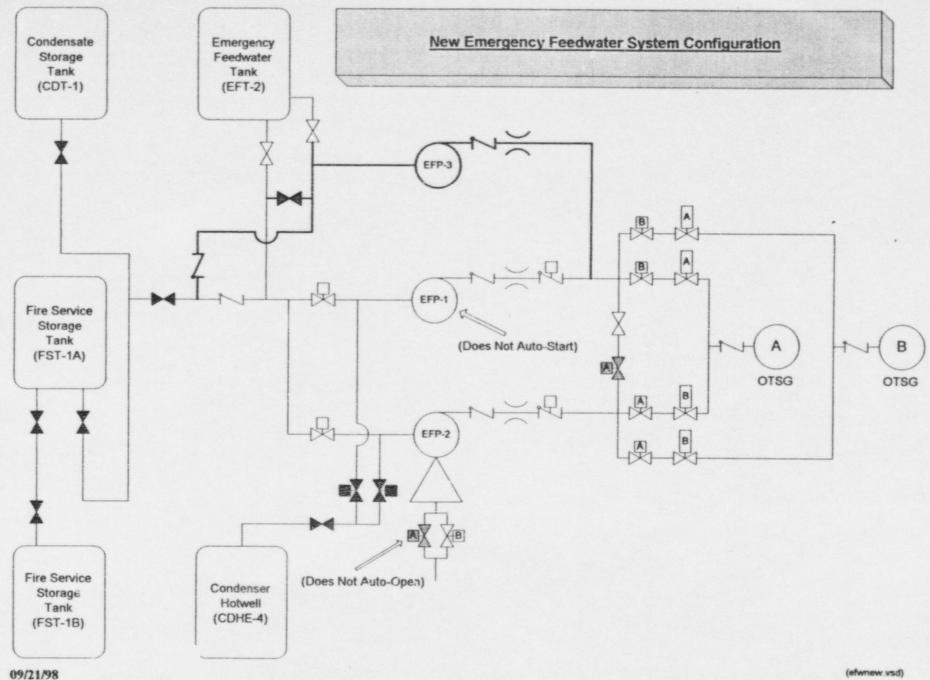
Design Features

- New Safety Related Emergency Feedwater Pump (EFP-3)
- New Safety Related Diesel Engine
- > Independent Class 1 Building
- > Fuel Oil Supply Allows 7 Day Operation Plus Testing
- All Required Indication and Control Power for Diesel Provided by Dedicated Class 1E Batteries Located in New Building
- AC-Independent Design Enhances SBO Coping Strategy
- > Applicable Codes and Standards
 - » Building NUREG 0800, Standard Review Plan
 - » Equipment Current Approved Codes and Standards (e.g., B31.1, 1967)



Design Features (continued)

- Pump Location Allows Suction from All Water Sources Except Condenser Hotwell. Administratively Controlled Inventory Exceeds the Requirement for Natural Circulation Cooldown
- EFP-1, EFP-2, FWP-7 can Transfer Water from the Hotwell to the Emergency Feedwater Tank / Condensate Storage Tank





Design Features (continued)

- Disposition of EFP-1
 - » Actuation Logic will be Transferred from EFP-1 to EFP-3
 - » Logic Prevents Concurrent Operation of EFP-3 and EFP-1
 - » EFP-1 will be Capable of Being Manually Selected
 - » EFP-1 can be Loaded on the "A" Diesel Generator When Capacity Allows
 - » EFP-1 will be Credited for Post-Seismic Cooldown
- Remove Auto-Open Signal from ASV-204



Industry Experience

- Diesel Driven Safety Related Emergency / Auxiliary Feedwater Pumps
 - » Byron Units 1 and 2
 - » Braidwood Units 1 and 2
 - » Fort Calhoun
- Other Diesel Driven Safety Related Pumps
 - » Byron Units 1 and 2 (Emergency Service Water)
 - » Braidwood Units 1 and 2 (Emergency Service Water)
 - » Surry Unit 1 (Emergency Service Water)
 - » Prairie Island Units 1 and 2 (Nuclear Service Water)



Overall Safety Benefits

- > Multiple, Diverse Emergency Feedwater Pumps
 - » EFP-2 is Steam Driven Technical Specification Required Pump
 - » EFP-3 is Diesel Driven Technical Specification Required Pump
 - » EFP-1 is Motor Driven Safety Grade Manual Pump
 - » FWP-7 is Defense-in-Depth
- Reduce EDG Load Management in EOPs
- > Reduce "A" EDG Auto-Connected Load



Overall Safety Benefits (continued)

- Operator Actions Eliminated
 - » Need to Cross-Tie EFP-2 to "A" Train (EFV-12) [OA9]¹
 - » Need to Place SWP-1A and RWP-2A in Pull-to-Lock and EFP-1 in Trip Defeat - [OA11]¹
 - » Need to Place EFP-1 in Trip Defeat Prior to LPI/EFP-1 Interlock (500 psig) - [OA15]¹
 - » Need to Manage Overfill Protection on Loss of "B" Battery
 - ¹ Operator Actions were Identified in the NRC Safety Evaluation Report for License Amendment No. 163.



- Required Technical Specification Revisions
 - Removal of License Amendment No. 163 Cycle 11 Specific Limitations
 - > Removal of Technical Specifications for EFP-1
 - > New Technical Specifications for New Equipment



■ Project Schedule

	1998											1999												
	J	F	M	IA	M	IJ		I	S	0	N	D	J	F	M	A	M	J	J	A	S	0	ND	
Design		1	1	1		1	-	-	+	-	-	-						-				1		
Design Review Board		1	1	1	1			1	1	1	1	1					1	1	-		1	1	1	
Design Review Board		1	1	1	1	1	1	1	1	=	1	1						1	-		1	1	1	
Licensing Submittal		1	1	1	1	1	1	i	1	1	.	i				i		1			1	1	1	
Construction		1	1	1	1	1	1	1	1	1								-				. !	1	
OPs Revised		1	1	1	1	1	1	-	1	1	-					1		1			1	1	1	
Simulator Modified		1	1	1	1	-	1	1	1	-					1	1	1	1	1	1	1	1	1	
EOP Validation			1	1	1	1	1	-	1	1	1		200		-			1	1	1	1	1	1	
EOP Training (First Cycle)		1	1	1	1	i	i	i	i	1	1				i	i				1	1	1	i	
Requested NRC Review & Approval			1	1	1	1	1	İ	1	1	1	-								1	1	1	1	
EOP Training (Second Cycle)			1	1	1	1	1	1	1	1	1				1	1	1	-			1	1	1	
Outage 11R Begins		1	1	1	1	1	1	1	1	1	1		1	1	1	-	1	1	1	1	-	1	1	
8							L		1				- 1	_1		1	1	1	1	1		1	1	

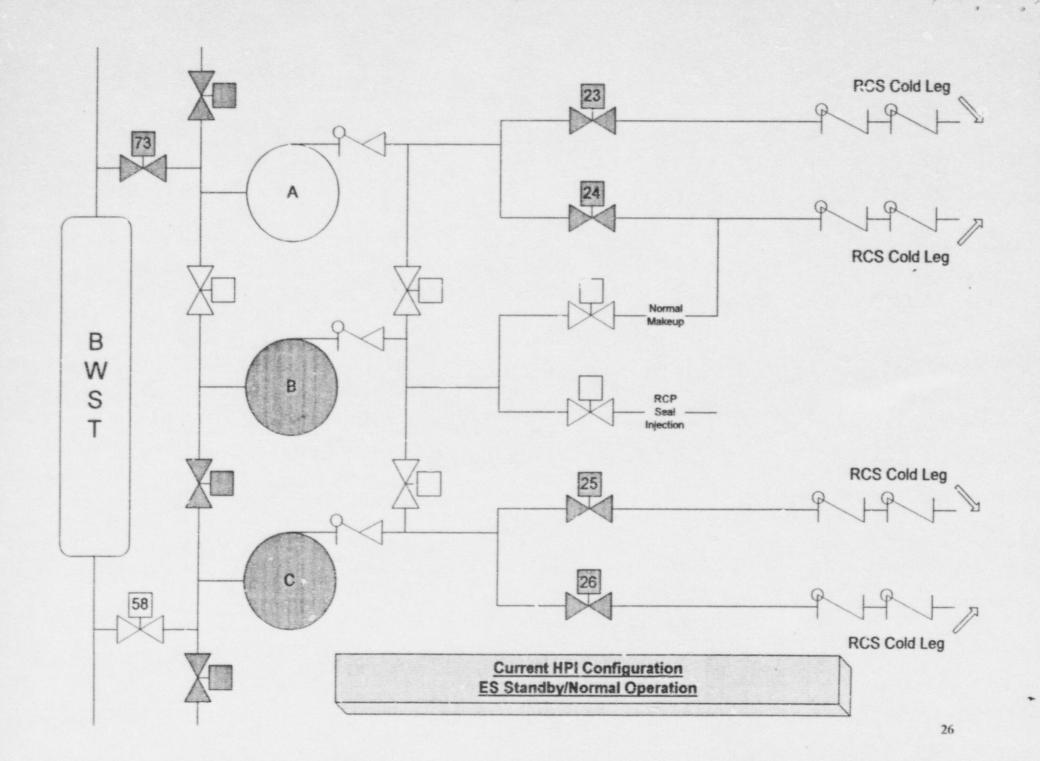


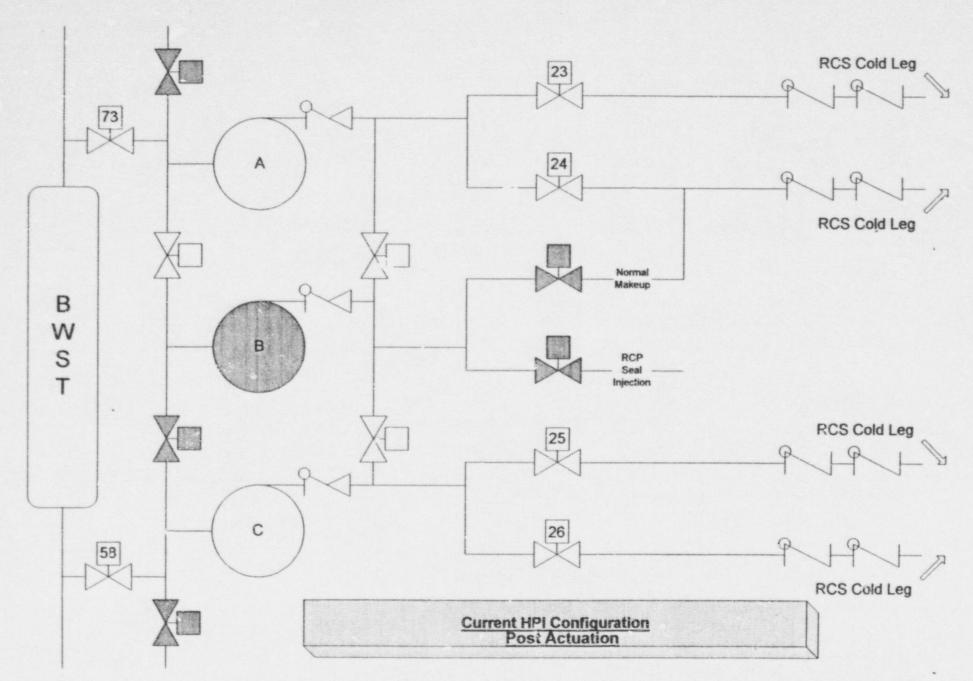
- HPI Configurations and Operation
 - > License Amendment No. 163
 - > SBLOCA
 - » Spectrum that Requires OTSG Cooling / Emergency Feedwater
 - > Operator Actions Required to Mitigate Accident
 - Elevated Peak Clad Temperature (PCT) for Cold Leg
 Pump Discharge (CLPD) Break

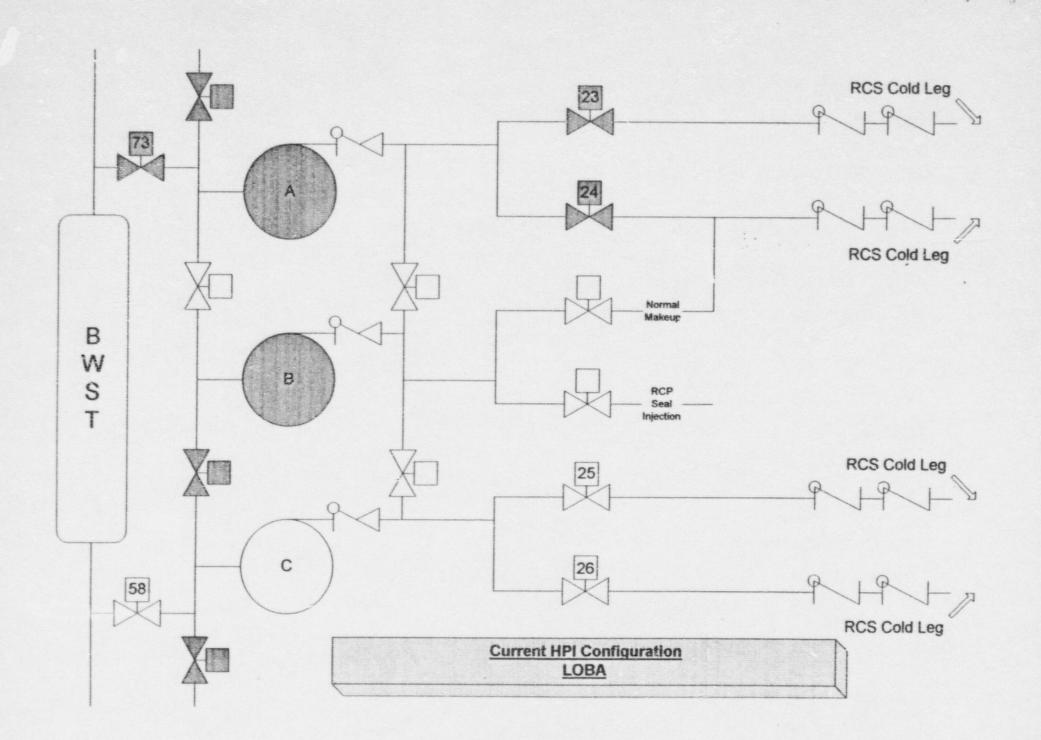


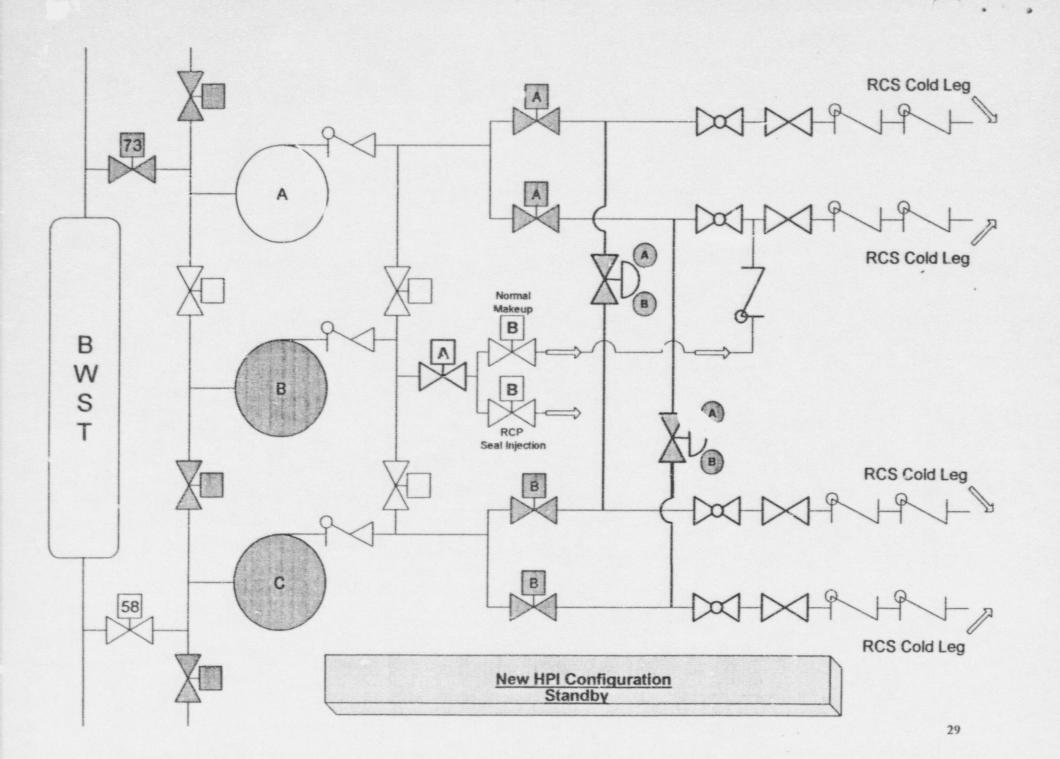
Operator Actions

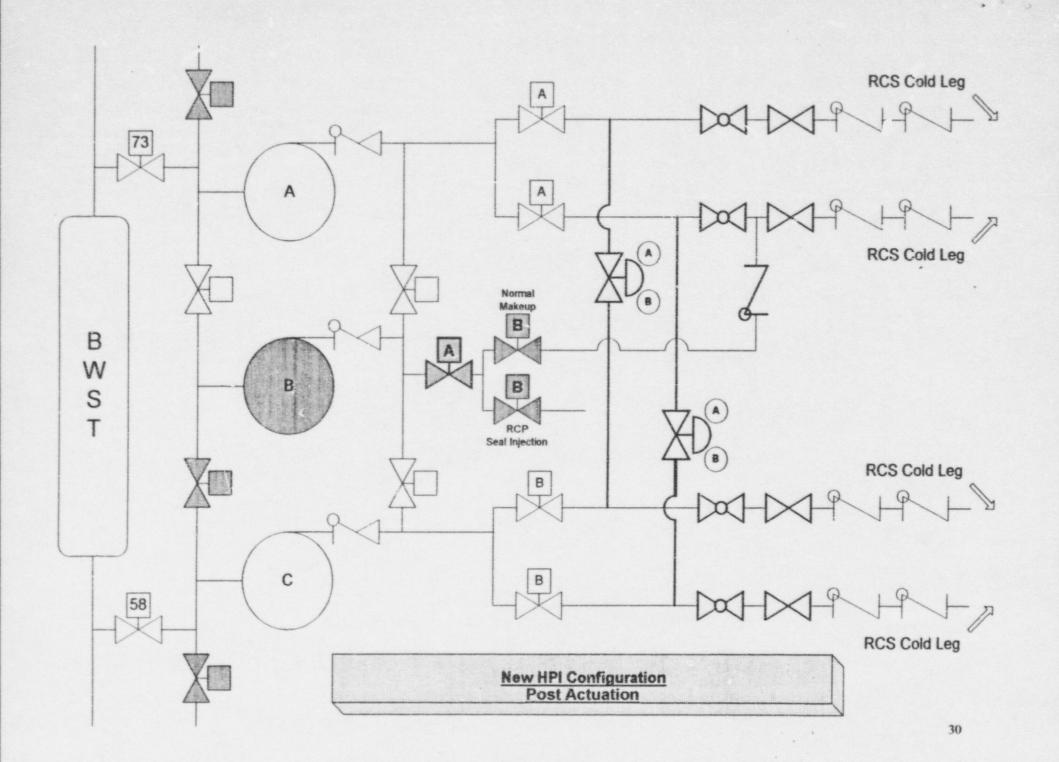
- > Trip RCPs
- > Ensure all 4 HPI Valves Open
 - » Includes Power Transfer
- > Isolate Normal Makeup
 - » Includes Power Transfer
- > Isolate Broken HPI Line
 - » Initial Evaluation
- > Monitor HPI Flow
 - » Subsequent Evaluation if OTSG Heat Transfer Upset
- > Isolate RCP Seal Injection
- > Raise OTSG Level to 95%

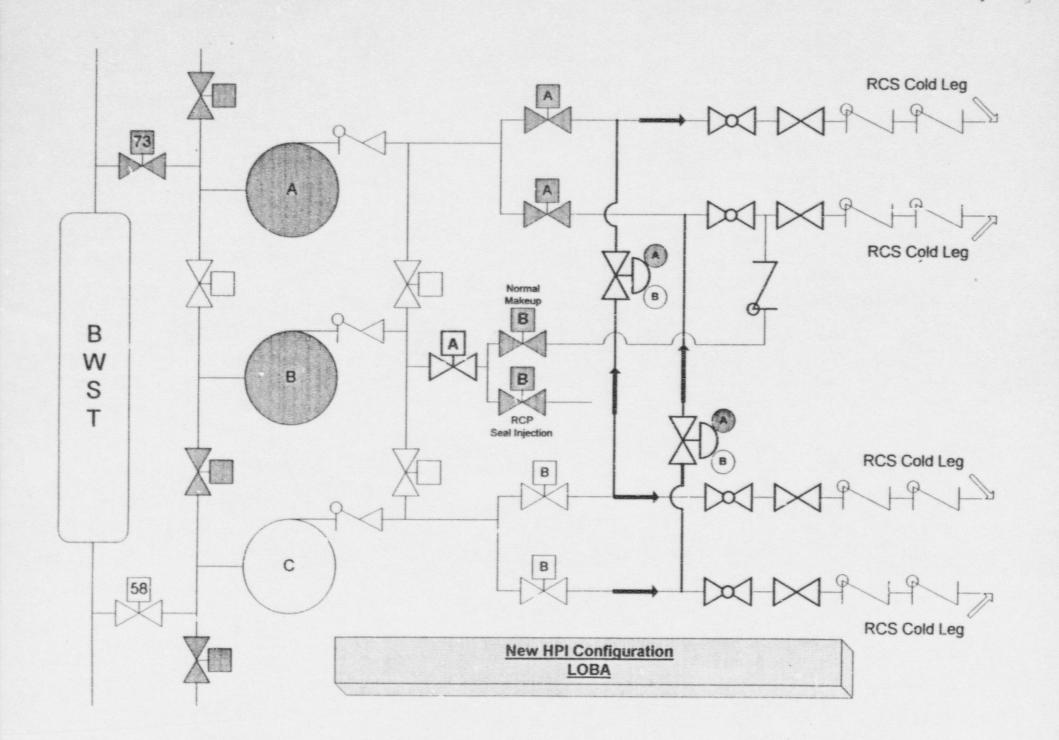














Design Alternatives

- > Detailed Evaluation of Alternatives
- Design Features
 - Cross-tie Lines Between HPI Lines to Ensure Flow in All Four Injection Lines
 - Isolation Valves in Cross-tie Lines to Limit Thermal Fatigue Cycles
 - Throttle Valves Downstream of Cross-ties to Balance Injection Line Flow and Limit Flow During a Postulated HPI Line Break
 - > Auto-Isolation of Normal Makeup and Seal Injection Flow
 - > Increased ES Actuation Setpoint
 - » Currently 1500 psig (nominal)
 - » Will be ~1600 psig
 - » Other Setpoints (RCS Trip and Bypass, ES Bypass) Adjusted Accordingly



- Required Technical Specification Revisions
 - > Add Surveillance Requirements for New Configuration
 - Revise Reactor Protection System and Engineered Safeguards Setpoints



Overall Safety Benefits

- > Reduced PCT (RELAP5)
- Reduced Operator Actions
 - » Eliminate Need to Open All 4 HPI Valves [OA3]¹
 - » Eliminate Need to Isolate RCP Seal Injection [OA4]1
 - » Eliminate Need to Isolate Normal Makeup
 - » Eliminate Need to Isolate Broken HPI Line [OA5]¹
 - » Eliminate Need to Periodically Evaluate HPI Line Break Criteria on Repressurization [OA17]¹
 - ¹ Operator Actions were Identified in the NRC Safety Evaluation Report for License Amendment No. 163.



- Overall Safety Benefits (continued)
 - > Brings CR-3 in Line with Other B&W Plants
 - » Peak Clad Temperature (PCT)
 - » Setpoints
 - » Design



■ Project Schedule

	1998												1999 JFMAMJJASOND											
	J	ï	M	IA	1	1.	J,	J	A S	S	0	N	D	J	F	M	A	M	J	J	A	S	0	ND
Design		-	1	-					-		-	BAS C		-		.			1	1	1	1		
Design Review Board			1	1	1	1	1	1	-	1	-	1		1	1	1	1			1	1	-		
Licensing Submittal		1	1	1	1	1	1	1	1	1	1	-	-	1	-	1	1			1	1	1		
Construction		1	1	1	1	1	-	1	1	1	1	1		1	1	1	1				-			1
EOPs Revised			1	1	-	1	-	1	1	-					1	1	1							
Simulator Modified		1	1	1	1	i	1	1	1	1				1	1	1	1					1		1
EOP Validation			-	1	-	1	1	1	1	1	1	-	1		4	-				1				
EOP Training (First Cycle)			1	1	1	1	1	-	i	1	1	1		1	1	1	1						1	i
Requested NRC Review & Approval			1	1	1	1	1	1	1	1	1	-		1	-	-							1	1
EOP Training (Second Cycle)				1	1	1	1	1	-	1	1	1		1	1	1	1						-	
Outage 11R Begins	1		1	-	1	1	1	1	1	1	1	1	-	1	ļ	1	1	1					8	1



Diesel Driven Emergency Feedwater Pump and HPI Upgrade Projects

Conclusion

- Net Safety Benefits
 - » Reduces Operator Burden in EOPs
 - » Reduces Worst Case SBLOCA PCT
 - » Increases EFW Redundancy/Diversity
 - » Eliminates EDG Load Management Requirements
 - » Eliminates EFW Cross-train Relationships
- > Meets NRC Commitments
- > Schedule Considerations