

DETAILS

I. Persons Contacted

Principal Licensee Employees

J. H. Goldberg, Group Vice President, Nuclear
E. Hill , Project Compliance Engineer
*S. M. Head, Lead Project Compliance Engineer
K. O'Gara, Project Compliance Engineer
R. R. Hernandez, Supervisor, Project Compliance
F. Alkov, Material Control Supervisor
*L. Dolan, Project Compliance Engineer
*R. Daly, Startup Manager
*A. G. Peterson, Startup
T. J. Jordan, Project QA Manager
*D. R. Keating, Operations QA General Supervisor
J. T. Westermeier, Project Manager
R. C. Arthurs, Project QA General Supervisor
*W. H. Kinsey, Plant Manager
S. M. Dew, Deputy Project Manager
R. L. Balcom, Reactor Operations Manager
M. A. Ludwig, Maintenance Manager

Other Personnel

Bechtel Power Corporation (Bechtel)

*J. B. Gatewood, Project QA Engineer
M. H. Alexander, Materials Manager
R. H. Medina, Lead QA Engineer
D. M. Stover, Construction Manager
J. E. Noxon, QA Engineer
*J. A. Brown, Project Services Superintendent

Ebasco Services, Inc. (Ebasco)

*J. E. Blackwood, Construction Superintendent
A. M. Cutrona, QA Manager
R. M. Zaist, Construction Manager
*R. G. Peck, Deputy Quality Program Site Manager

Westinghouse

A. Hogarth, Site Manager

In addition to the above personnel, the NRC inspectors held discussions with various operations, engineering, technical support, maintenance, and administrative members of the licensee's staff.

*Denotes those individuals attending the exit interview conducted on December 2, 1985.

2. Site Tours

The NRC inspector made plant tours, both independently and with licensee and contractor personnel. These tours were to assess the protection of in-place plant equipment, fire protection, plant status and to observe ongoing construction testing and maintenance activities. The areas toured included: Unit 1; Mechanical and Electrical Auxiliary Building (MEAB), Fuel Handling Building, Diesel Generator Building, and Reactor Containment Building; Unit 2; Reactor Containment Building, MEAB, Turbine Building and Fuel Handling Building, Balance of Plant (BOP); and the Emergency Cooling Pump Building, Security Building and warehouses.

The NRC inspector observed control room staffing, startup prerequisite testings and plant maintenance. Operations and QA support personnel were observed in the plant tracing systems for training.

No violation or deviations were identified.

3. Licensee Action on Previous Inspection Finding:

(Closed) Unresolved Item 498/499-8408-01 Storage/Maintenance of Nuclear Steam Supply System (NSSS) Equipment

This unresolved item concerns how differences between vendor manuals and "Westinghouse NSSS Component Receiving and Storage Guidelines Manual" are resolved and documented. A joint group composed of Westinghouse site personnel and Bechtel Storage and Maintenance Technical Support (SMTS) Group was formed. The purpose of this joint group was to reverify the maintenance requirements for all NSSS equipment received at the time of the review under PO 4000/8000, the NSSS supplier, and identify and correct any discrepant conditions.

This review was accomplished by comparing the Westinghouse NSSS Component Receiving and Storage Guidelines Manual against the applicable manufacturer's technical manuals. Any conflict between these documents was resolved and were incorporated into the Westinghouse NSSS Component Receiving and Storage Guidelines Manual. The review encompassed approximately 1,767 individual sub-assemblies/components and identified no anomalies which could adversely affect the integrity of the equipment. However, the storage level requirement for a semi-automatic refueling machine was changed from Level C to Level B. Westinghouse concurrence on the results of this evaluation has been received by the licensee.

The Westinghouse NSSS Equipment Maintenance Review Form for documenting Maintenance Action Card (MAC) system maintenance requirements and recording Westinghouse concurrence has been proceduralized in WPP/QCI 28.0, Revision 11, "Maintenance of Material Equipment."

General Surveillance Report S-B-0208 dated March 12, 1985 verified completion of this review program and resolution of the discrepancy discussed above.

The NRC inspector has reviewed the above documentation and licensee actions. Based on the results of the rereview it appears that there was no violation of NRC requirements. This unresolved item is closed.

(Closed) Open Item 498/499-8503-01 Draft Preoperational Test Procedure Review, 1-DJ-P-01 "125V DC Battery System, Channel I"

This open item concerns comments by the NRC inspector on the safety-related Preoperational Test Procedure 1-DJ-P-01, "125V DC Battery system, Channel I." This draft procedure was provided to the NRC at the same time it was sent to the Joint Test Group (JTG) and was the first safety-related test procedure. The procedure was inadequate. An extensive list of comments was provided to the licensee.

The licensee has completely revised procedure 1-DJ-P-01, FSAR Section 14.2.12.2.20 and 21 (1E Battery acceptances criteria) and their method of test draft test procedure review. The NRC inspector has reviewed the issued Preoperational Text Procedure 1-DJ-P-01, Revision 1, and found it acceptable with all NRC comments provided on the draft procedure being acceptably resolved. This open item is closed.

(Closed) Unresolved Item 498/499-8415-01 Startup Organization

This unresolved item concerns differences between licensees actual startup organization and that described in the STP Final Safety Analysis Report (FSAR) Section 14.2, Startup Administrative Instruction (SAI) 1.02, "Startup Organization." In mid-1984, the startup organization was undergoing a major reorganization, including a new startup manager. The FSAR has since been revised (Amendment 48) to reflect the current organization and assigned responsibilities. SAI 1.02 was reissued as SAI 2, "Startup Organization," Revision 0, and also describes the current organization. The NRC inspector has reviewed the above documentation and concluded that no violation of NRC requirements occurred. This unresolved item is closed.

4. Preoperational Test Procedure Review

During this inspection the NRC inspectors reviewed the following preoperational test procedures:

- 1-EW-P-01 Essential Cooling Water System Performance - Train 1A
- 1-EW-P-02 Essential Cooling Water System Performance - Train 1B
- 1-EW-P-03 Essential Cooling Water System Performance - Train 1C

- 1-RM-P-01 Reactor Makeup Water System
- 1-WL-P-01 Reactor Coolant System Drain Tank
- 1-DJ-P-01 125V DC Class 1E System Channel 1
- 1-DJ-P-02 125V DC Class 1E System Channel 2
- 1-DJ-P-03 125V DC Class 1E System Channel 3
- 1-DJ-P-04 125V DC Class 1E System Channel 4

The listed preoperational test procedures were reviewed to determine if the contents were in accordance with the FSAR, Regulatory Guide 1.68 and the licensee's administrative procedures.

The NRC inspectors determined that the procedures were technically adequate to accomplish their objectives.

The NRC inspector also reviewed the following related instructions and procedures. The Startup Administrative Instructions (SAI), Desk Top Instructions (DN), and Quality Assurance Implementing Procedures (QAIP) were reviewed to verify that they are technically adequate to accomplish the stated purpose.

- SAI-1 Startup Manual Use and Control
- SAI-2 Startup Organization
- SAI-3 Startup Quality Assurance Plan
- SAI-10 Indoctrination Training and Certification of Test Personnel
- SAI-17 Prerequisite Testing
- SAI-18 Preoperational Testing
- SAI-19 Acceptance Testing
- SAI-22 Conduct of Startup Operations
- DN-85-001 Desk Top Instructions provides guidelines for preparing/developing preoperational test procedures
- QAIP-1.0 Organization
- QAIP-2.0 Indoctrination and Training

- QAIP-2.1 Certification Requirements for Site Surveillance Personnel
- QAIP-4.0 Standard Format for Writing and Controlling Procedures and Manuals
- QAIP-5.0 In-Process Document Control
- QAIP-7.0 Records Review
- QAIP-8.0 Audits
- QAIP-9.0 Surveillance
- QCIP-1.0 Receiving Inspection
- QCIP-2.0 Operations Quality Control Call Log
- QCIP-3.0 Inspection Reporting
- QCIP-4.0 Area/Systems Turnover to Nuclear Plant Operations Department
- QCIP-5.0 Quality Control Document Reviews
- QCIP-6.0 Inspection Planning
- QCIP-8.0 Certification of Inspection Examination and Test Personnel

Within the areas examined, the NRC inspector found the instructions and procedures adequate.

No violations or deviations were identified.

5. Allegation Follow-up: 4-85-A-080

A general allegation was telephoned to Region IV regarding mismanagement of the South Texas Project Safeteam. The caller indicated that a letter with specific information and selected documents would be forth coming. No such letter was received and attempts to contact the individual by telephone and certified mail were unsuccessful (see NRC Inspection Report 50-498/85-19 and 50-499/85-17). A NRC inspector learned that the same allegations by the same individual, a former Safeteam investigator, were received by the Safeteam several months prior to the NRC notification. Since the allegations involved the Safeteam itself the licensee contracted for a third party investigation by an experienced consulting firm. An NRC inspector reviewed the licensee's Safeteam investigation, the independent

consultant's investigation and the information provided to the NRC and found all actions taken were reasonable and acceptable. The results of that investigation was the voluntary termination of general foreman in August 1984. Based on the various reviews by independent organizations, including the NRC and the inability to acquire additional information from the alleged, this allegation is closed.

No violation or deviations were identified.

6. Exit Interview

An exit interview was conducted on December 2, 1985, with those personnel denoted in paragraph 1 of this report. During the exit interview, the NRC inspectors summarized the scope and findings of this inspection.