



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20565-0001

OMAHA PUBLIC POWER DISTRICT

DOCKET NO. 50-285

FORT CALHOUN STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 187
License No. DPR-40

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Omaha Public Power District (the licensee) dated October 3, 1997, as supplemented by letter dated May 18, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9810210219 981019
PDR ADOCK 05000285
P PDR

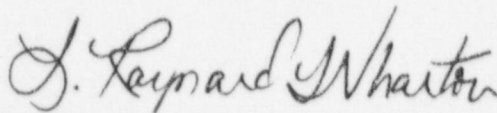
2. Accordingly, Facility Operating License No. DPR-40 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-40 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 187, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance to be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



L. Raynard Wharton, Project Manager
Project Directorate IV-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: October 19, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 187

FACILITY OPERATING LICENSE NO. DPR-40

DOCKET NO. 50-285

Revise Appendix "A" Technical Specifications as indicated below. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

3-62

3-62a

INSERT

3-62

3-62a

3-62b

3.0 SURVEILLANCE REQUIREMENTS

3.9 Auxiliary Feedwater System

Applicability

Applies to periodic testing requirements of the turbine-driven and motor-driven auxiliary feedwater pumps.

Objective

To verify the operability of the auxiliary feedwater (AFW) system and its ability to respond properly when required.

Specifications

- (1) The position of valves necessary to ensure auxiliary feedwater flow to the steam generators shall be verified by a monthly inspection. Anytime maintenance is performed on the auxiliary feedwater system which alters valve alignments, an operator shall check that the affected AFW system valves are properly aligned, to ensure AFW flow to the steam generators, and a second operator shall independently verify proper valve alignment.
- (2) The operability of the motor-driven auxiliary feedwater pump and the steam turbine-driven auxiliary feedwater pump shall be confirmed at least monthly.
- (3) The operability of auxiliary feedwater pumps' steam generator level regulating valves HCV-1107A, HCV-1107B, HCV-1108A, HCV-1108B, and auxiliary feedwater cross-tie valve HCV-1384 shall be confirmed in accordance with the Inservice Testing Program.
- (4) Verify the proper alignment of the required AFW flow paths by verifying flow from the emergency feedwater storage tank to each steam generator prior to raising the reactor coolant temperature (T_c) above 300°F whenever the unit has been in MODE 4 and/or 5 for greater than 30 days.
- (5) On a refueling frequency:
 - a. Verify that each automatic valve in the flow path actuates to its correct position upon receipt of each auxiliary feedwater actuation test signal.
 - b. Verify that each auxiliary feedwater pump starts as designed automatically upon receipt of each auxiliary feedwater actuation test signal.

3.0 SURVEILLANCE REQUIREMENTS

3.9 Auxiliary Feedwater System (Continued)

Basis

The basic safety related function of the auxiliary feedwater (AFW) system is to provide an alternate source of feedwater to the steam generators in the event of a loss of main feedwater. The auxiliary feedwater system is designed so that a single active failure coincident with a loss of offsite power will not prevent a safe plant shutdown.

Specification 3.9(1)

The valve position verifications performed monthly will confirm the availability of an auxiliary feedwater flow path to the steam generators. Following AFW system maintenance, the affected valves are verified to be in the correct position.

Specification 3.9(2)

The testing of the auxiliary feedwater pumps every month will verify their operability by recirculating water to the emergency feedwater storage tank.

Proper functioning of the AFW steam turbine admission valve and starting of the feedwater pump will demonstrate the integrity of the steam driven pump. Verification of correct operation will be made both from instrumentation within the main control room and direct visual observation of the pumps.

Specification 3.9(3)

Operating AFW system cross-tie valve HCV-1384 and regulating valves HCV-1107A, HCV-1107B, HCV-1108A and HCV-1108B one at a time in accordance with the Inservice Testing Program will confirm operability of the valves.

Specification 3.9(4)

This surveillance ensures that the AFW system is properly aligned by verifying flow from the emergency feedwater storage tank to each steam generator. The required flow paths are through valve HCV-1384 and valves HCV-1105/HCV-1106 using FW-6. The surveillance is performed prior to raising the reactor coolant temperature (T_c) above 300°F whenever the reactor coolant temperature has been below 210°F for greater than 30 days. OPERABILITY of the AFW flow paths must be verified before sufficient core heat is generated that would require the operation of the AFW system during a subsequent shutdown.

3.0 SURVEILLANCE REQUIREMENTS

3.9 Auxiliary Feedwater System (Continued)

Basis (Continued)

Specification 3.9(5)

This surveillance ensures that the AFW pumps will start, and automatic valves will reposition, in the event of any accident or transient that generates an AFAS. The surveillance demonstrates that AFW pumps FW-6 and FW-10 start, and valves HCV-1107A/B and HCV-1108A/B actuate to their correct position automatically, on a test signal. The test is accomplished by utilizing overlapping tests to verify that the instrumentation, valves and motor driven pump are OPERABLE with the steam turbine-driven pump tested separately to verify its OPERABILITY. The refueling frequency is based on the need to perform this surveillance under the conditions that apply during a unit outage and the potential for an unplanned transient if the surveillance were performed with the reactor at power. This frequency is acceptable, based on the design reliability and operating experience of the equipment.

References

- (1) USAR, Section 9.4
- (2) Technical Specification 2.5