

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below.

(1) Maximum Power Level

Southern Nuclear is authorized to operate the facility at reactor core power levels not in excess of 3565 megawatts thermal (100 percent power) in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 103, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

DELETED → The Surveillance Requirements (SRs) contained in the Appendix A Technical Specifications and identified below are not required to be performed immediately upon implementation of Amendment No. 96.

SRs 3.8.1.9 and 3.8.4.7 shall be successfully demonstrated prior to the first entry into MODE 4 following the seventh refueling.

(3) Initial Startup Test Program (Section 14, SER)*

DELETED → Any changes to the initial Test Program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

(4) Emergency Planning (Section 13.3, SER and SSERs 2, 3, 4, and 5)

DELETED → In the event that the NRC finds that the lack of progress in completion of the procedures in the Federal Emergency Management Agency's final rule, 44 CFR Part 350, is an indication that a major substantive problem exists in achieving or maintaining an adequate state of emergency preparedness, the provisions of 10 CFR Section 50.54(s)(2) will apply.

* The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

(5) Steam Generator Tube Rupture (Section 15.6.3, SER and SSER 3)

By March 1, 1988, GPC shall submit for NRC review a revised plant-specific steam generator tube rupture analysis based on the Westinghouse Owner's Group generic resolution, which includes radiological consequence analyses, analysis of steamline static load in the event of overfill, and justification that systems and components credited in the analysis to mitigate accident consequences are safety related.

DELETED

(6) DELETED

(7) NUREG-0737 Items

- a. Compliance with NUREG-0737, Item II.F.2 (Section 4.4.8, SER and SSERs 1 and 4)

In accordance with NUREG-0737, Item II.F.2, GPC shall submit the proposed reactor vessel level instrumentation system (RVLIS) report by June 1, 1987.

- b. Supplemental Report on Safety Parameter Display System (Section 18.2, SSER 6)

GPC shall submit by March 1, 1988, a supplemental report on the safety parameter display system as discussed in Section 18.2 of SSER 6.

- c. Supplemental Summary Report on Detailed Control Room Design Review (Section 18, SSER 5)

GPC shall submit by March 1, 1988, a Supplemental Summary Report on the detailed control room design review discussing:

1. the final results of the remaining control room surveys (ambient noise; illumination; heating, ventilation, and air conditioning; plant safety monitoring system computer survey; automatic turbine supervisory instrumentation computer survey; and communications) and the resolution of any human engineering discrepancies (HEDs) resulting from these surveys
2. a complete assessment of cumulative and interactive effects of the HEDs
3. the completed review of annunciator nuisance alarms and modifications to minimize nuisance alarms and the number of annunciator windows lit during normal operations

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- 4. documentation showing tradeoff analyses and other information used in resolving HEDs
- 5. the methodology by which control room changes were to be factored into the operators' training program
- 6. procedures that incorporate human factors review into the design process for future control room modifications

DELETED

(8) Zinc Coating of Diesel Fuel Oil Storage Tanks (Section 9.5.4.2, SSER 4)

Prior to restart following the first refueling, GPC shall (1) replace the zinc coating in the diesel generator fuel oil storage tanks with a coating which does not contain zinc or (2) by March 1, 1988 provide an acceptable justification to the staff that the present fuel oil storage tank zinc-based coating will not affect the operability and reliability of the diesel generators over the life of the plant as specified in IE Circular 77-15.

If option (1) is chosen, GPC shall provide the NRC with a modification status report 30 days before the expiration of the license condition.

DELETED

(9) Alternate Radwaste Facility (Section 11.4, SSERs 3 and 4)

Prior to restart following the first refueling, the ventilation exhaust of the alternate radwaste facility shall be modified to exhaust through HEPA filters already installed in the auxiliary building HVAC system.

GPC shall provide the NRC with a modification status report 30 days before the expiration of the license condition.

(10) Additional Conditions

The Additional Conditions contained in Appendix D, as revised through Amendment No. 102, are hereby incorporated into this license. Southern Nuclear shall operate the facility in accordance with the Additional Conditions.

- D. The facility requires exemptions from certain requirements of 10 CFR Part 50 and 10 CFR Part 70. These include (a) an exemption from the requirements of 10 CFR 70.24 for two criticality monitors around the fuel storage area, (b) an exemption from the requirements of Paragraph III.D 2(b)(ii) of Appendix J of 10 CFR 50, the testing of containment air locks at times when containment integrity is not required, and (c) a schedular exemption from 10 CFR 50.34(b)(2)(i) as it pertains to GDC 2, 61, and 62 of Appendix A to 10 CFR 50 for the spent fuel pool racks for the time period before the racks contain irradiated fuel. The special circumstances regarding exemptions b and c are identified in Sections 6.2.6 and 9.1.2 of SSER 5, respectively.

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DELETED

H. Reporting to the Commission

DELETED → Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, Southern Nuclear shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within twenty-four (24) hours to the NRC Operations Center via the Emergency Notification System with written follow-up within 30 days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).

- I. The Owners shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- J. This license is effective as of the date of issuance and shall expire at midnight on January 16, 2027.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Attachment 1 - DELETED
- 2. Appendix A - Technical Specifications
- 3. Appendix B - Environmental Protection Plan
- 4. Appendix C - Antitrust Conditions
- 5. Appendix D - Additional Conditions

Date of Issuance: March 16, 1987

**FACILITY OPERATING LICENSE
NO. NPF-81**

VOGTLE UNIT 2

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified or incorporated below.

(1) Maximum Power Level

Southern Nuclear is authorized to operate the facility at reactor core power levels not in excess of 3565 megawatts thermal (100 percent power) in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 81, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

The Surveillance Requirements (SRs) contained in the Appendix A Technical Specifications and listed below are not required to be performed immediately upon implementation of Amendment No. 74. The SRs listed below shall be successfully demonstrated prior to the time and condition specified below for each:

- DELETED
- a) SRs 3.8.1.8, 3.8.1.11 and 3.8.1.13 shall be successfully demonstrated prior to the first entry into MODE 4 following the fifth refueling outage.
 - b) SR 3.8.1.9 shall be successfully demonstrated prior to the first entry into MODE 4 following the sixth refueling outage.
 - c) SR 3.8.1.20 shall be successfully demonstrated at the first regularly scheduled performance after implementation of this license amendment.

(3) Additional Conditions

The Additional Conditions contained in Appendix D, as revised through Amendment No. 80, are hereby incorporated into this license. Southern Nuclear shall operate the facility in accordance with the Additional Conditions.

AND

- D. The facility requires exemptions from certain requirements of 10 CFR Part 50 and 10 CFR Part 70. These include (a) an exemption from the requirements of 10 CFR 70.24 for two criticality monitors around the fuel storage area, (b) an exemption from the requirements of Paragraph III.D.2(b)(ii) of Appendix J of 10 CFR 50, the testing of containment air locks at times when containment integrity is not required, and (c) an exemption from the schedule requirements of 10 CFR 50.33(k)(1) related to availability of funds for decommissioning the facility. The special circumstances regarding exemptions b and c are identified in Sections 6.2.6 and 22.5 of SSER 8, respectively.

DELETED

An exemption was previously granted pursuant to 10 CFR 70.24. The exemption was granted with NRC materials license No. SNM-1981, issued July 13, 1988, and relieved GPC from the requirement of having a criticality alarm system. GPC and Southern Nuclear are hereby exempted from the criticality alarm system provision of 10 CFR 70.24 so far as this section applies to the storage of fuel assemblies held under this license.

These exemptions are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security. The exemptions in items b and c above are granted pursuant to 10 CFR 50.12. With these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

- E. Southern Nuclear shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans are entitled: "Vogtle Electric Generating Plant Unit 1 and Unit 2 Physical Security and Contingency Plan" (which contains Safeguards Information protected under 10 CFR 73.21) with revisions submitted through September 12, 1996, and "Vogtle Electric Generating Plant Guard Training and Qualification Plan," with revisions submitted through March 13, 1996. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.
- F. GPC shall comply with the antitrust conditions delineated in Appendix C to this license.
- G. Southern Nuclear shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility, as approved in the SER (NUREG-1137) through Supplement 9 subject to the following provision:

Southern Nuclear may make changes to the approved fire protection program without prior approval of the Commission, only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

H. Reporting to the Commission

DELETED → Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, Southern Nuclear shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within twenty-four (24) hours to the NRC Operations Center via the Emergency Notification System with written follow-up within 30 days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).

- I. The Owners shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- J. This license is effective as of the date of issuance and shall expire at midnight on February 9, 2029.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Thomas E. Murley, Director
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Appendix A - Technical Specifications
- 2. Appendix B - Environmental Protection Plan
- 3. Appendix C - Antitrust Conditions
- 4. Appendix D - Additional Conditions

Date of Issuance: March 31, 1989