

**Northeast  
Nuclear Energy**

Rope Ferry Rd. (Route 156), Waterford, CT 06385

Millstone Nuclear Power Station  
Northeast Nuclear Energy Company  
P.O. Box 128  
Waterford, CT 06385-0128  
(860) 447-1791  
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The Northeast Utilities System

OCT 9 1998

Docket No. 50-336  
B17497

Re: 10CFR50.71(a)

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555


Millstone Nuclear Power Station, Unit No. 2  
Facility Operating License No DPR-65  
Monthly Operating Report

In accordance with the reporting requirements of Technical Specification Section 6.9.1.7 for Millstone Unit No. 2, enclosed, in Attachment 1, is the monthly operating report for the month of September 1998.

There are no regulatory commitments contained within this letter.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

  
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J. A. Price  
Director, Millstone Unit No. 2

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PDR ADDCK 05000336  
R PDR

- cc: H. J. Miller, Region I Administrator  
D. G. McDonald, Jr., NRC Senior Project Manager, Millstone Unit No. 2  
D. P. Beaulieu, Senior Resident Inspector, Millstone Unit No. 2  
E.V. Imbro, Director, Millstone ICAVP Inspections  
S. Dembek, NRC Project Manager, Millstone Unit No. 1

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Attachment 1

Millstone Nuclear Power Station, Unit No. 2

Facility Operating License No. DPR-65  
Monthly Operating Report

October 1998

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-336  
UNIT: Millstone Unit 2  
DATE: 10/01/98  
COMPLETED BY: S. Stark  
TELEPHONE: (860) 447-1791  
EXT: 4419

MONTH: September 1998

DAY	AVG. DAILY POWER LEVEL (MWe-Net)	DAY	AVG. DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	--
16	0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

UNIT NAME: Millstone Unit 2  
 DATE: 10/01/98  
 COMPLETED BY: S. Stark  
 TELEPHONE: (860) 447-1791  
 EXT: 4419

OPERATING STATUS

- 1. Docket Number 50-336
- 2. Reporting Period September 1993
- 3. Utility Contact S. Stark
- 4. Licensed Thermal Power (MWt): 2700
- 5. Nameplate Rating (Gross MWe): 909
- 6. Design Electrical Rating (Net MWe): 870
- 7. Maximum Dependable Capacity (Gross MWe): 901.63
- 8. Maximum Dependable Capacity (Net MWe): 870.63
- 9. If Changes Occur in Capacity Ratings (Items Number 4 Through 8) Since Last Report, Give Reasons:

N/A

Notes: Items 22 and 23 cumulative are weighted averages. Unit operated at 2560 MWTH prior to its uprating to its current 2700 MWTH power level.

- 10. Power Level To Which Restricted, If any (Net MWe): 0
- 11. Reasons For Restrictions, If Any: NRC Category III Facility; NRC Confirmatory Order requiring implementation of an independent corrective action verification program; NRC order requiring a third-party review of the employee concerns program at Millstone 2; design basis verification response pursuant to 10CFR50.54(f).

This Month Yr.-To-Date Cumulative

	This Month	Yr.-To-Date	Cumulative
12. Hours In Reporting Period	720.0	6551.0	199559.0
13. Number Of Hours Reactor Was Critical	0.0	0.0	121911.7
14. Reactor Reserve Shutdown Hours	0.0	0.0	2205.5
15. Hours Generator On-Line	0.0	0.0	116611.9
16. Unit Reserve Shutdown Hours	0.0	0.0	468.2
17. Gross Thermal Energy Generated (MWH)	0.0	0.0	300862506.4
18. Gross Electrical Energy Generated (MWH)	0.0	0.0	98709460.0
19. Net Electrical Energy Generated (MWH)	-1622.4	-15483.9	94603606.3
20. Unit Service Factor	0.0	0.0	58.4
21. Unit Availability Factor	0.0	0.0	58.7
22. Unit Capacity Factor (Using MDC Net)	0.0	0.0	55.3
23. Unit Capacity Factor (Using DER Net)	0.0	0.0	54.6
24. Unit Forced Outage Rate	100.0	100.0	27.0

- 25. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):  
 Shutdown at the time of this report

- 26. If Unit Shutdown At End Of Report Period, Estimated Date of Startup: To be determined

- 27. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-336  
 UNIT NAME: Millstone Unit 2  
 DATE: 10/01/98  
 COMPLETED BY: S. Stark  
 TELEPHONE: (860) 447-1791  
 EXT: 4419

REPORT MONTH: August 1998

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	License Event Report #	System Code <sup>4</sup>	Component Code <sup>6</sup>	Cause & Corrective Action to Prevent Recurrence
98-01	03/07/96	S/F	720	B/D	4	N/A	N/A	N/A	Scheduled: Continued mid cycle surveillance testing from previous month.  Forced: Continued from previous month. NRC Category III facility; NRC Confirmatory Order requiring independent corrective action verification; NRC order requiring third party review of Millstone Station employee concerns program; design basis verification for response to NRC pursuant to 10CFR50.54(f).

<sup>1</sup>F: Forced  
 S: Scheduled

<sup>2</sup>Reason  
 A - Equipment Failure (Explain)  
 B - Maintenance or Test  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & License Examination  
 F - Administrative  
 G - Operational Error (Explain)  
 H - Other (Explain)

<sup>3</sup>Method  
 1 - Manual  
 2 - Manual Scram  
 3 - Automatic Scram  
 4 - Continued from Previous Month  
 5 - Power Reduction (Duration = 0)  
 6 - Other (Explain)

<sup>4</sup>IEEE Standard 805-1984, "Recommended Practices for System Identification in Nuclear Power Plants and Related Facilities"

<sup>6</sup>IEEE Standard 803A-1983, "Recommended Practices for Unique identification in Power Plants and Related Facilities - Component Function Identifiers"

REFUELING INFORMATION REQUEST

1. Name of the facility: Millstone Unit 2
2. Scheduled date for next refueling outage: Second Quarter of 2000
3. Scheduled date for restart following refueling: First Quarter of 1999 (Note - The current shutdown is not a refueling outage. This date represents the expected startup date from the current shutdown.)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?  
Yes. There are 29 Technical Specification Change Requests or license amendments which are required to be approved by the NRC prior to startup. As of September 30, 1998, 22 requests have been sent to the NRC, 12 have been approved, and 11 have been implemented. The 7 remaining changes to be submitted are: 1) Boron Precipitation Hot Leg Injection USQ, 2) Long Term Cooling CST Makeup via Fire Water USQ, 3) HPSI Pull to Lock - Tech Spec Change, 4) Passive failure - ECCS pipe after a DBA - USQ, 5) Cable Tray Separation - USQ, 6) SGTR - Rad Consequences - USQ, and 7) Loss of Normal Feed - USQ.
5. Scheduled date(s) for submitting licensing action and supporting information:  
November 30, 1998
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: Small Break LOCA analysis changes and Large Break LOCA analysis changes.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:  
In Core: (a) 0 In Spent Fuel Pool: (b) 1085

NOTE: These numbers represent the total Fuel Assemblies and Consolidated Fuel Storage Boxes (3 total containing the fuel rods from 6 fuel assemblies) in these two (2) Item Control Areas.

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present storage capacity: 1306 storage locations

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9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming present license capacity:

2002, Spent Fuel Pool Full, Core offload capacity is reached.

2006, Core Full, Spent Fuel Pool Full.