



Point Beach Nuclear Plant
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NPL 98-0860

10 CFR 50.54(f)

October 14, 1998

Document Control Desk
U.S. NUCLEAR REGULATORY COMMISSION
Mail Station P1-137
Washington, DC 20555-0001

Ladies/Gentlemen:

DOCKETS 50-266 AND 50-301
CLARIFICATION OF
RESPONSE TO GENERIC LETTER 97-06
DEGRADATION OF STEAM GENERATOR INTERNALS
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

On March 30, 1998, Wisconsin Electric Power Company (WE), licensee for the Point Beach Nuclear Plant (PBNP), submitted its response to Generic Letter 97-06, "Degradation of Steam Generator Internals," dated December 30, 1997. The Generic Letter (GL) emphasized the importance of performing comprehensive examinations of steam generator internals to ensure steam generator tube structural integrity is maintained in accordance with the requirements of 10 CFR 50, Appendix B.

The WE response outlined the steam generator visual inspection program at PBNP and described changes required in the program. There were five new commitments identified in our letter dated March 30, 1998. The purpose of this letter is to rescind a statement in the following commitment that pertains to tube support plate ligament erosion-corrosion and cracking in our PBNP Unit 2 steam generators. The statement to be rescinded is indicated in italics:

"Eddy current inspection is not applicable for the tube support plates due to the trifoil broached hole design. The only presently available inspection is visual. PBNP will continue to perform secondary side tube bundle inspections every five years. The procedures will be revised to specifically look for signs of tube support plate degradation in addition to tube fouling prior to the next Unit 2 refueling outage (U2R23). *Additionally, an inspection of the top tube support plate will be performed during the next refueling outage for signs of separated ligaments.*"

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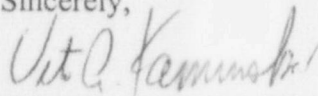
As indicated in our March 30, 1998, submittal, the PBNP Unit 2 steam generators are Westinghouse Model Delta 47 with broached trifoil stainless steel tube support plates (TSPs). The upcoming Unit 2 refueling outage (U2R23), scheduled to begin on December 4, 1998, will be the first refueling outage since replacement of the steam generators. The top TSP has flow holes instead of elongated slots in the tube lane region. Flow holes are used to strengthen the top TSP for U-bend support. If initial drilling produced a separated ligament, an evaluation of the effect on U-bend support is recommended.

Extensive visual inspections were performed of the PBNP Unit 2 steam generators during fabrication and following installation. The post-installation inspections were reviewed, and no evidence of separated ligaments exists on either top TSP. The entire tube lane region of the top TSP of each steam generator was inspected in January 1997 and the flow holes were determined to be evenly spaced.

Initial drilling did not produce separated ligaments. Therefore, no further inspections or evaluation of the top TSPs for PBNP Unit 2 are required. WE has decided that there is no need to perform this inspection during the upcoming outage. Accordingly, the commitment to perform this inspection is retracted. Since all trifoil tube support plates are stainless steel, there is no additional visual inspection recommended by the manufacturer.

If you have any questions or require additional information, please contact us.

Sincerely,



Vito A. Kaminskas

Manager,
Regulatory Services & Licensing

FAF/dms

cc: NRC Resident Inspector
PSCW

NRC Regional Administrator
NRR Project Manager