NRC Form (9-83)	366	LICENSEE EVENT REPORT (LER)								U.S. NUCLEAR REGULATORY COMMISS APPROVED OMB NO. 3150-0194 EXPIRES 8/31/8:								
FACILITY	NAME (1)			-			-		KET NUMBER	(2)	PAGE (3)						
		NOR'	TH AN	NA POWER	STATION	. UNIT 2				0 5 0 0	0131319	1 OF 016						
TITLE (4)						N DURING N	MAINTE	NANCE										
EVE	ENT DATE			LER NUMBER	-	REPORT DAT	-			FACILITIES INVOL	LVED (8)							
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POWE		20.402(6)				20.405(c) 50.35(c)(1)		X	50,73(a)(2)(iv) 50,73(a)(2)(v)		73.71(c)							
LEVE (10)	L .	010		1.405(a)(1)(ii)	-	50.36(c)(2)		-	50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
1107		LUIU	-	.405(a)(1)(iii)	Y	50.73(e)(2)(i)			50.73(a)(2)(viii)	(A)								
		20.405(a)(1)(iv)				50.73(a)(2)(ii)			80.73(a)(2)(viii)									
			20).405(a)(1)(v)		50.73(a)(2)(iii)			50.73(a)(2)(x)	3(a)(2)(x)								
		*********	-		L	ICENSEE CONTACT	FOR THIS	LER (12)										
NAME											TELEPHONE NUM	BER						
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		G. I	E. Ka	nne, Stat	ion Manag	ger				71013	819141-	15/1/5/1						
				COMPLET	E ONE LINE FOR	EACH COMPONEN	T FAILURE	DESCRIBE	C IN THIS REPO	RT (13)								
CAUSE	SYSTEM	COMP	ONENT	MANUFAC TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPROS							
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				SUPPLEA	MENTAL REPORT	EXPECTED (14)				EVERATE	MONTH	DAY YEAR						

At 1104 hours on May 4, 1987, with Unit 2 at 100 percent power, an inadvertent partial train "B" Engineered Safety Features (ESF) actuation of the Containment Depressurization System occurred. As a result of prompt operator action and the specific equipment configuration, an actual Containment Spray did not occur. This event is reportable pursuant to 10CFR50.73(a)(2)(i)(B) and 10CFR50.73(a)(2)(iv). A four hour report was made in accordance with 10CFR5f.72(b)(2)(ii).

The actuation occurred when a ground was introduced into the Solid State Protection System during valve maintenance. As a corrective action, the Control Room operators verified that the actuation was spurious, performed the necessary actions to enable resetting the actuation signal, and successfully reset the actuation signal. To prevent recurrence of similar events, applicable procedures will be reviewed and revised as necessary to include additional cautions and notes and require the use of insulated tools when working in switches, boxes, etc.

No significant safety consequences resulted from this event because an actual Containment Spray did not occur. Also, equipment actuations were verified to have actuated per design and were subsequently restored to their normal configuration. The health and safety of the general public was not affected at any time during this event.

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YES III yes complete EXPECTED SUBMISSION DATE!

ABSTRACT IL imit to 1400 spaces i.e. approximately fifteen single space typewritten

NRC Form 366A (9-83)									ME NO 3150-0104								
FACILITY NAME (1)			DOCKET NUMBER (2)								R NUMBER 16		PAGE (3)				
										YEAR		SEQUENTIAL NUMBER		REVISION			
NORTH ANNA	POWER STATION.	UNIT 2	0	5	10	0	0	3 3	19	8 8	_	0 0 1 3	-	010	0 2	OF	0 16

1.0 Description of Event

At 1104 hours on May 4, 1987, with Unit 2 at 100 percent power, an inadvertent partial train "B" Engineered Safety Features (ESF) (EIIS System Identifier) actuation of the Containment Depressurization System (EIIS System Identifier) occurred. The actuation occurred when a ground was introduced into the Solid State Protection System (SSPS) (EIIS System Identifier) during valve maintenance. As a result of prompt operator action and the specific equipment configuration, an actual Containment Spray did not occur. This event is reportable pursuant to 10CFR50.73(a)(2)(iv) and 10CFR50.73(a)(2)(i)(B) Technical Specification 3.0.3 was entered when both Train "B" Recirculation Spray pumps were placed in "Pull to Lock". A four hour report was made in accordance with 10CFR50.72(b)(2)(ii).

Prior to this event, maintenance was being performed on MOV-QS-200B, a Train "B" Quench Spray (QS) valve (EIIS System Identifier . Component Identifier) located on the suction side of the Train "B" QS pump (2-QS-P-1B).

During the performance of a limit switch adjustment MOV-QS-200B, the SSPS output relay K643 was energized. Review of station drawings revealed that a limit switch on MOV-QS-200B provided an interlock to a test circuit associated with relay K643. This limit switch allows K643 to be tested only when MOV-OS-200B is closed and is normally at a 120VAC potential relative to the SSPS. The coil of K643 is in series with the limit switch. Therefore, grounding the limit switch energizes the relay. When a partially insulated screwdriver shaft came in contact with the interlocking limit switch, the circuit was completed, and relay K643 was energized because the tip of the screwdriver was at ground potential. When relay K643 was energized, it actuated two other output relays, K643XB1 and K643XB2. As a result, the following ESF actuations occurred:

Service Water (EIIS System Identifier) flow was initiated to the four Recirculation Spray Heat Exchangers (EIIS System Identifier , Component Identifier).

Approximately 800 gallons of borated water was slucied from the RWST to the Containment Recirculation Spray Sump (EIIS System Identifier, Component Identifier) through an idle Train "B" QS pump. (NOTE: This quantity of water posed no operational problems and was easily removed by normal containment sump pump operation.)

The Safeguards Ventillation System dampers (EIIS System Identifier , Component Identifier) shifted to redirect flow

from the normal vent path to the iodine filter.

NRC Form 366A (9-83)	LICENSEE E	EVENT	REPOR	RT (LE	R) 1	EX	ст с	CON	NTIN	U	ATIO	N		U.S	API	PROVED OMB NO. 3150-0104 PRESS 8/31/88									
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Service Water flow was isolated to the Unit 2 Component Cooling Water Heat Exchangers (EIIS System Identifier, Component Identifier).

The 2J Emergency Bus Stub Bus Breaker (EIIS System Identifier , Component Identifier) tripped. (There were no loads on the stub bus.)

Upon receiving an annunciator alarm, the Control Room Operators varified normal indica ion of containment pressure on the four containment pressure protection channels and unsuccessfully attempted to reset the actuation signal. To prevent inadvertent starting of the Inside and Outside Recirculation Spray pumps after a time delay and subsequent pump damage, the operators placed both Train "B" Recirculation Spray Pumps in the "Pull to Lock" position. After the operators notified the electricians to stop working on MOV-QS-200B, relay K643 was successfully reset. It is believed that earlier attempts to reset relay K643 were unsuccessful because the uninsulated screwdriver was in contact with the interlocking limit switch at the time the reset was attempted.

Equipment was restored to the normal configuration by approximately 40 minutes after the ESF actuation was initiated. Actuations that occurred were verified to have occurred per design (see Attachment 1).

2.0 Significant Safety Consequences and Implications

No significant safety consequences resulted from this event because an actual Containment Spray did not occur. Also, equipment actuations were verified to have actuated per design and were subsequently restored to their normal configuration. The health and safety of the public were not affected at any time during this event.

3.0 Cause of the Event

The partial ESF actuation occurred when a ground was introduced into the Solid State Protection System (SSPS) and actuated an output relay. The ground resulted when a partially insulated metal screwdriver came in contact with a limit switch which is normally at a 120 VAC potential relative to the SSPS and is in series with the coil of the output relay.

U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/88 DOCKET NUMBER (2) FACILITY NAME (1) LER NUMBER (6) PAGE (3) QUENTIAL NUMBER YEAR NORTH ANNA POWER STATION, UNIT 2 014 OF 016 0 |5 |0 |0 |0 |3 |3 |9 |8 |8 |-- 010

0 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

4.0 Immediate Corrective Action

As an immediate corrective action, the Control Room operators verified normal indication of containment pressure on the four containment pressure protection channels and reset the ESF actuation signal.

5.0 Additional Corrective Action

Following the event, a Significant Event Review meeting was held with station management and the individuals involved in the event. a result of this meeting, the following additional corrective actions were performed:

The maintenance procedure was deviated to insert a caution to insulate the adjustment tool and maintenance was satisfactorily completed on MOV-QS-200B at 2147 hours.

The amount of water slucied from the RWST was quantified and the flowpath was verified to be through the idle quench spray pump 2-QS-P-1B and down a 4 inch orifice line to the recirculation spray sump.

The following additional corrective actions will be performed:

Operation with the Recirculation Spray Heat Exchanger filled with Service Water is still being evaluated.

Engineering will perform a review of the adequacy of the circuitry design and determine how many other similar circuits exist.

6.0 Actions to Prevent Recurrence

To prevent recurrence, seven other MOV's with similar interlock design characteristics have been identified. A review has also been initiated to identify any more MOV's and/or devices where safeguards relays or other safeguard equipment could be actuated. identification, appropriate procedures will be reviewed and revised to include notes/cautions with respect to vital bus power and relay actuation and the potential results of relay actuation.

Additionally, procedures will be revised to require the use of insulated tools and/or other insulating methods when working switches, boxes, etc.

NRC Form 366A (9-83) LICENSEE EVENT REP	PORT (LER) TEXT CONTIN	IUATION		MB NO: 3150-0104 //88
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		YEAR SEO	UENTIAL REVISION	
NORTH ANNA POWER STATION, UNIT 2	0 5 0 0 0	818 - 0	1013 - 010	0 5 OF 0 6

7.0 Similar Events

Previous inadvertent ESF actuations during maintenance/testing are provided below:

Unit 1	Unit 2
84-002-00	84-009-00
84-020-00	87-013-00
85-023-00	87-014-00
86-017-00	87-016-00
87-012-00	87-018-00
87-022-00	

8.0 Additional Information

Unit 1 was in Mode 1 during this event and was not affected.

Operation of the Recirculation Spray Heat Exchangers filled with Service Water has been determined to be reportable pursuant to 10CFR50.73(a)(2)(ii)(B) and will be reported in LER N1/N2-88-016-00.

NRC Form 386A (9-8-3) LICENSEE EVENT	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED EXPIRES: 8/								
FACILITY NAME (1)	DOCKET NUMBER (2)	4.6	ER NUMBER (6)	PAGE (3)					
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NORTH ANNA POWER STATION, UNIT	2 0 5 0 0 0 3 3 9	8 8 -	0 10 1 3 - 0 10	0 16 0 0 0 16					

TEXT (If more space is required, use additional NRC Form 366A's) (17)

ATTACHMENT 1

EQUIPMENT ACTUATION LIST

RELAY	FUNCTION VI	ERIFIED	EXPLANATION FOR N
K643	Closes MOV-SW-208B Opens MOV-QS-201B Opens MOV-SW-201D Starts 2-RS-P-2B	Y Y Y N	Pump Placed in PTL by Operator
	Opens MOV-SW-205B Energizes K643XB1 Energizes K643XB2	Y Y Y	
K643XB1	Closes MOV-SW-210B Closes MOV-SW-214B Energizes Relay 3F Energizes Relay 3N Trips Breaker 15J12 (Stub Bus)	N N N Y Y	De-energized per Operating Procedure De-energized per Operating Procedure Could not Verify, ESF Signal Reset
K643XB2	Closes SOV-HV-228-2 Closes SOV-HV-115A-2	Y Y	
3F	Interlocks MOV-SW-205B Interlocks MOV-SW-208B Interlocks MOV-SW-210B Interlocks MOV-SW-214B Interlocks MOV-SW-201D Blocks Trip 2-RS-P-2B	N N N N N	Could not Verify, ESF Signal Reset Could not Verify, ESF Signal Reset
3N	Block Close MOV-RS-2561 Interlock MOV-QS-210B Trips SPDS File Block Close MOV-RS-2551	N Y	Could not Verify, ESF Signal Reset Could not Verify, ESF Signal Reset Could not Verify, ESF Signal Reset



VIRGINIA ELECTRIC AND POWER COMPANY

P. O. BOX 402

MINERAL, VIRGINIA 23117

May 25, 1988

U. S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, D.C. 20555 Serial No. N-88-021 NO/DEQ: nih Docket No. 50-339

License No. NPF-7

Dear Sirs:

The Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Unit 2.

Report No. LER 88-003-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very truly yours,

G. E. Kane Station Manager

Enclosure

cc: U. S. Nuclear Regulatory Commission 101 Marietta Street, N. W. Suite 2900 Atlanta, Georgia 30323

> Mr. J. L. Caldwell NRC Senior Resident Inspector North Anna Power Station

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