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On April 10, 1988, Quad Cities Unit Two was shutdown for the end of cycle 9 refueling and maintenance outage. On April 13, 1988 at 1630 hours, it was determined that the measured combined leakage rate from all penetrations and valves, excluding the Main Steam Isolation Valves, exceeded the Technical Specifications (3.7.A.2) limit of 293.75 SCFH (0.60La). This was identified while local leak rate testing the 2-220-58B and 2-220-62B feedwater check valves.

The failure mode of the penetrations and valves is not known at this time since the testing and repair of these items is not complete. A supplemental report will address leak rate testing and repairs when this is completed. This report is submitted to comply with the requirements of 10CFR50.73(a)(2)(ii).

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)					Page (3)		
		Year	11/1	Sequential Number	Revision Number				
Quad Cities Unit Two	0 5 0 0 0 2 6 5	818	-	01017	- 010	012	OF	0	

PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor - 2511 MWt rated core thermal power. Energy Industry Identification System (EIIS) codes are identified in the text as [XX].

EVENT IDENTIFICATION: The leak rate from all valves and penetrations, excluding the Main Steam Isolation Valves, is in excess of the Technical Specification Limit of 0.60 La (293.75 SCFH) as defined in the Technical Specifications.

A. CONDITIONS PRIOR TO EVENT:

Unit: Two	Event Date: April 13, 1988	Event Time: 1630
Reactor Mode: 1	Mode Name: Shutdown	Power Level: 00%

This report was initiated by Deviation Report D-4-2-88-019

Shutdown Mode(1) - In this position, a reactor scram is initiated, power to the control rod drives is removed, and the reactor protection trip systems have been deenergized for 10 seconds prior to permissive for manual reset.

B. DESCRIPTION OF EVENT:

On April 10, 1988, Unit Two was shutdown for the end of cycle nine refueling and maintenance outage. On April 13, 1988 at 1630 hours while performing refueling outage local leak rate testing, the 2-220-588 and 2-220-628 (feedwater [SJ] check valves [V]) volume was shown to have a leakage rate of 890.1 Standard Cubic Feet Per Hour (SCFH). This event is indicative of excessive leakage of one or both of these valves. Since the leakage rate was 890.1 SCFH, the Technical Specification (3.7.A.2) limit of 293.75 SCFH (0.60 La) combined leakage from all valves and penetrations except Main Steam Isolation Valves [SV, ISV], was exceeded.

The failure mode of this and other penetrations [PEN] and valves to be tested during the refuel outage is not known at this time, since the testing and repair of these items is not complete. A supplemental report shall be submitted when all valves and penetrations are repaired and tested.

C. APPARENT CAUSE OF EVENT:

This report is being submitted to comply with the requirements of IOCFR50.73(a)(2)(ii) which requires the reporting of any event or condition that results in the condition of the nuclear power plant, including its principle safety barrier, being seriously degraded.

The cause of the valve and penetration leakage will not be determined until all repairs have been completed and tested. A supplemental report documenting these repairs and any corrective actions taken will then be issued.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER	HUMBER	(6)	1999 A.		Page (3)				
		Year	11/1	Sequential Number	11/1	Revision Number					
Quad Cities Unit Two	0 5 0 0 0 2 6 5	8 8	-	0 0 1 7	-	010	013	OF	01		

D. SAFETY ANALYSIS OF EVENT:

The safety consequences of this event were minimal since local leak rate testing is a conservative method of measuring containment leakage. During accident conditions, the actual leakage would be less than that determined by local leak rate testing since some lines would be pressurized with water instead of air and some Non-Primary Containment Isolation Valves would perform to isolate the primary containment.

E. CORRECTIVE ACTIONS:

No corrective action has been taken as of this date. A supplemental report will be submitted listing the necessary repairs and any corrective actions taken to reduce the combined leakage below the required limit.

F. PREVIOUS EVENTS:

- 254/87-016 Leak rate from all valves and penetrations on Unit One in excess of Technical Specification limit.
- 265/86-014 Leak rate from all valves and penetrations c.. Unit Two in excess of Technical Specification limit.
- 254/86-001 Leak rate from all valves and penetrations on Unit One in excess of Technical Specification limit.
- 265/85-007 Leak rate from all valves and penetrations on Unit Two in excess of Technical Specification limit.
- 254/84-002 Leak rate from all valves and penetrations on Unit One in excess of Technical Specifications.

3. COMPONENT FAILURE DATA:

Component failure data is not yet available since repairs have not been completed. The supplemental report will document the failure data.



Commonwealth Edison Quad Cities Nuclear Power Station 22710 206 Avenue North Cordova, Illinois 61242 Telephone 309/654-2241

RLB-88-151

April 26, 1988

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Reference: Quad-Cities Nuclear Power Station Docket Number 50-265, DPR-30, Unit Two

Enclosed please find Licensee Event Report (LER) 88-007, Revision 00, for Quad-Cities Nuclear Power Station.

This report is submitted in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(11), which requires the reporting of any event or condition that results in the condition of the nuclear power plant, including its principle safety barrier, being seriously degraded.

Respectfully,

COMMONWEALTH EDISON COMPANY QUAD-CITIES NUCLEAR POWER STATION

R. L. Bax Station Manager

RLB/MSK/e

Enclosure

cc: I. Johnson R. Higgins INPO Records Center NRC Region III

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