

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) River Bend Station	DOCKET NUMBER (2) 0 5 0 0 0 4 5 8	PAGE 5 1 OF 0 4
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TITLE (4)
Unqualified Tubing Installed Due to Procedural Error

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																									
0 2	0 5	8 6	8 6	0 1	7 0	0 3	0 6	8 6			0 5 0 0 0																																									
<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9)</td> <td style="width:15%;">1</td> <td colspan="10">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50. (Check one or more of the following): (11)</td> </tr> <tr> <td rowspan="5">POWER LEVEL (10) 0 2 0</td> <td>20.402(b)</td> <td>20.402(a)</td> <td>90.73(a)(2)(iv)</td> <td>73.71(a)</td> </tr> <tr> <td>20.402(a)(1)(i)</td> <td>90.38(a)(1)</td> <td>90.73(a)(2)(v)</td> <td>73.71(a)</td> </tr> <tr> <td>20.402(a)(1)(ii)</td> <td>90.38(a)(2)</td> <td>90.73(a)(2)(vi)</td> <td>OTHER (Specify in Abstract below and in Text, NRC Form 308A)</td> </tr> <tr> <td>20.402(a)(1)(iii)</td> <td>X 90.73(a)(2)(i)</td> <td>90.73(a)(2)(vii)(A)</td> <td></td> </tr> <tr> <td>20.402(a)(1)(iv)</td> <td>90.73(a)(2)(ii)</td> <td>90.73(a)(2)(vii)(B)</td> <td></td> </tr> <tr> <td>20.402(a)(1)(v)</td> <td>90.73(a)(2)(iii)</td> <td>90.73(a)(2)(viii)</td> <td></td> </tr> <tr> <td>20.402(a)(1)(vi)</td> <td>90.73(a)(2)(iv)</td> <td>90.73(a)(2)(ix)</td> <td></td> </tr> </table>												OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50. (Check one or more of the following): (11)										POWER LEVEL (10) 0 2 0	20.402(b)	20.402(a)	90.73(a)(2)(iv)	73.71(a)	20.402(a)(1)(i)	90.38(a)(1)	90.73(a)(2)(v)	73.71(a)	20.402(a)(1)(ii)	90.38(a)(2)	90.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)	20.402(a)(1)(iii)	X 90.73(a)(2)(i)	90.73(a)(2)(vii)(A)		20.402(a)(1)(iv)	90.73(a)(2)(ii)	90.73(a)(2)(vii)(B)		20.402(a)(1)(v)	90.73(a)(2)(iii)	90.73(a)(2)(viii)		20.402(a)(1)(vi)	90.73(a)(2)(iv)	90.73(a)(2)(ix)	
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LICENSEE CONTACT FOR THIS LER (12)

NAME Liz Thompson - Nuclear Engineer	TELEPHONE NUMBER AREA CODE: 5 0 4 6 3 5 - 6 0 9 4
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

At 1605 on 02/05/86 with the unit at 20 percent power, it was documented that non-seismically qualified tubing was installed in the Penetration Valve Leakage Control System (PVLCS). The tubing was installed to bypass solenoid operated valves that were not needed for the skid mounted design configuration of the PVLCS compressors. The root cause of the condition was determined to be a procedural deficiency. Immediate action was taken to declare both divisions of PVLCS inoperable and initiate a plant shutdown per Technical Specifications 3.6.1.10 and 3.0.3. The tubing was promptly removed and the procedure revised to correct the deficiency. Acceptable alternate designs were installed on both divisions by 0815 on 02/06/86. There was no effect on the public health and safety.

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TEXT: If more space is required, use additional NRC Form 388A (1/17)

Reported Condition

On 02/04/86 at approximately 1500 with the unit at 20 percent power, a design system engineer performed a post-installation design walkdown of Emergency Modification Request (MRE) 86-0001 on the Penetration Valve Leakage Control System (PVLCS). The modification was performed in order to bypass with tubing solenoid operated valves that were not needed for the skid mounted design configuration of the PVLCS compressors. It was observed during the walkdown that the tubing installed under the subject MRE which was prepared on 02/02/86 at approximately 1800 may not have been seismically qualified. A review of the MRE and discussions with the engineers involved confirmed this observation.

Investigation

A subsequent engineering review of the MRE and the PVLCS design bases defined in the Safety Analysis Report revealed that the tubing should have been designed to Seismic Category I requirements. The root cause of the event was found to be a procedural deficiency relating to the MRE process. The MRE, as a mechanism for approving design changes, was established on 01/31/86 via Interim Procedure Change 3-006-2-3 to Design Control Procedure NPE-3-006, Revision 2. The subject MRE was the first one to be processed under the new procedure change. Approval of an MRE required the engineers to use engineering judgement to determine if a change was acceptable. Under the existing procedure at the time, a full evaluation would be

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completed within 30 days. In addition, no documented Unreviewed Safety Question Determination (USQD) was required by the procedure prior to installation of the modification. Based on these procedure deficiencies, the engineers, acting within the direction of their procedures, approved the MRE.

There was only one other MRE approved using the same design review process, MRE 86-0002. That MRE was reviewed and found to be an acceptable design change as documented under Modification Request (MR) 86-0196.

Corrective Action

Both divisions of PVLCS were declared inoperable at 1605 on 02/05/86 and a plant shutdown initiated per Technical Specification 3.6.1.10 and 3.0.3. MR 86-0203 was issued on 02/05/86 to install an acceptable alternate design. The design change on one division was completed on 02/05/86 at 2155 placing the unit in a seven day limiting condition for operation (LCO) instead of a shutdown. The other division was completed on 02/06/86 at 0815 cancelling the LCO. On 02/06/86, the Director of Nuclear Plant Engineering issued memorandum NuPE-86-202 to all Design System Engineers directing them to perform a documented USQD on all MREs prior to work. The Design Control Procedure NPE-3-006 was revised on 02/21/86 via Interim Procedure Change 3-006-2-4 to require performance of a USQD prior to design approval.

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TEXT (if more space is required, use additional NRC Form 366A's) (17)

Safety Consequences

The capability of the tubing to withstand a design basis seismic event is unknown, it can be postulated that the safety function of the affected air charging capability of the PVLCS may have been adversely affected. No actual safety consequences resulted from the condition reported here and the health and safety of the public was not endangered.



GULF STATES UTILITIES COMPANY

RIVER BEND STATION POST OFFICE BOX 220 ST. FRANCISVILLE, LOUISIANA 70775

AREA CODE 504 835-6094 348-8651

March 6, 1986
RBG-23305
File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

River Bend Station - Unit 1
Docket No. 50-458

Please find enclosed Licensee Event Report No. 86-017 for River Bend Station - Unit 1. This report is submitted pursuant to 10CFR50.73.

Sincerely,

Eddie R Grant
for J. E. Booker
Manager-Engineering,
Nuclear Fuels & Licensing
River Bend Nuclear Group

JEB
JEB/TFP/DRG/BEH/ebm

cc: U.S. Nuclear Regulatory Commission
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Arlington, TX 76011

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