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50-324

Distribution
Docket File
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Mr. E. E. Utley
Senior Executive Vice President
Power Supply and Engineering
and Construction
P. O. Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

Subject: Development of Risk-Based Inspection Guide - Brunswick Steam Electric Plant, Units 1 and 2

The NRC has a program for producing plant-specific inspection guidance for resident inspectors, utilizing the risk insights from a plant's Probabilistic Risk Assessment (PRA). The Risk-Based Inspection Guide (RIG) for a particular plant contains information about its dominant accident sequences, the importance of the various plant systems that contribute to these accident sequences, and the more risk-significant failure modes of the important systems. Any plant-specific vulnerabilities are highlighted for the resident. In addition, inspection checklists are provided for the most risk-significant failure modes.

In order to produce a RIG for Brunswick, we will need the following information to supplement what we have in your PRA:

1. Indexes for normal, abnormal and emergency operating procedures.

2. Mode 1 normal operating procedures, abnormal operating procedures and emergency operating procedures for the systems listed in Attachment A.

3. If not included in the normal operating procedures, the Mode 1 system line-up checklists for systems listed in Attachment A.

4. System descriptions (typically from operator training materials) for systems 15, 16, 17, and 18 on Attachment A.

5. 11" x 17" piping and instrument diagrams listed in Attachment B.

OFO1
1/1

If you would like to discuss or clarify any of the requested items, please contact Steve Long at (301) 492-3162.

Thank you for your cooperation in our efforts to make our inspection efforts more efficient and effective.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, CMB clearance is not required under P.L. 96-511.

Sincerely,

/s/

Bart C. Buckley, Senior Project Manager
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachments:
As stated

| OFC | :LA:PD21:DRPR | :PM:PD21:DRPR | :D:PD21:DRPR | : | : | : |
|---|---|---|---|---|---|---|
| NAME | :PAnderson:jaw | :BBuckley | :GEAndersam | : | : | : |
| DATE | :9/21/88 | :9/21/88 | :9/21/88 | : | : | : |

Mr. E. E. Utley
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Brunswick Steam Electric Plant
Units 1 and 2

cc:

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ATTACHMENT A

Systems Analyzed in BSEP PRA for Which

Additional Information is Requested

1. High Pressure Coolant Injection
2. Reactor Core Isolation Cooling
3. Control Rod Drive (CRD) Hydraulic
4. Automatic Depressurization
5. Core Spray
6. Residual Heat Removal
7. Standby Liquid Control
8. Emergency Diesel Generators
9. AC Power
10. DC Power
11. Emergency Core Cooling System Actuation
    a) HPCI Actuation to F001, F006
    b) RCIC Actuation to F013, F045
    c) Actuation to RHR valves F015A, F015B
    d) LPCI Actuation to C002A, C002B, C002C, C002D
    e) Actuation Signal to E21-F005A, E21-F005B
    f) CSS Actuation to E21-C001A, C001B
12. Service Water
13. Reactor Building Closed Cooling Water
14. Heating, Ventilating and Air Conditioning
15.* Condensate
16.* Feedwater
17.* Reactor Protection
18.* Instrument Air

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* Not modeled in PRA.

ATTACHMENT B

BSEP-PRA/RIG EFFORT DRAWINGS REQUIRED

DUE TO MINOR ILLEGIBILITY OR NON-INCLUSION IN PRA

| Drawing No. | Title |
|---|---|
| D-2523, Sheet 1 | Reactor Building Piping Diagram High Pressure Coolant Injection System, Sheet 1, Unit No. 2 |
| D-2523, Sheet 2 | Same, Sheet 2 |
| D-2521-1A | Main Steam System |
| D-2521-1C | Feedwater System |
| D-2040-1B | Condensate Supply Grav. Unit 2 |
| F-4073 | Standby Gas Treatment |
| D-2529, Sheet 1 | Reactor Building - Piping Diagram Reactor Core Isolation Cooling System, Sheet 1, Unit No. 2 |
| D-2529, Sheet 2 | Same, Sheet 2 |
| D-2516, Sheet 1A | Reactor Building Piping Diagram Control Rod Drive Hydraulic System Sheet 1A, Unit No. 2 |
| D-2516, Sheet 1B | Same - Sheet 1B |
| D-2517-2B | Instrument Air 70-75 PSIG |
| D-7029-2A | Instrument Air 30 PSIG |
| F-P-50015-7 | Reactor Protection System |
| D-2520, Sheet 3A | Reactor Building Piping Diagram Nuclear Steam Supply System Sh. 3A, Unit No. 2 |
| D-2516, Sheet 1A | Reactor Building-Piping Diagram Control Rod Drive Hydraulic System Sheet 1A, Unit No. 2 |
| D-2516, Sheet 1B | Same-Sheet 1B |
| F-3043 | Units No. 1 & 2, Key One Line Diagram, 230KV, 24KV, & 4160 Volt Systems |

| Drawing No. | Title |
|---|---|
| F-3044 | Units No. 1 & 2 Key One Line Diagram 480 Volt System |
| F-3006 | Unit No. 2 Single Line Diagram 125/250 Volt DC System Distribution Switchboard 2A & 2B |
| D-20041-1 Sheet 1 | Piping Diagram-Service Water System Sheet 1 - Unit No. 1 |
| D-20041-2 | Same - Sheet 2 |
| D-20041-3 | Same - Sheet 3 |
| D-2034-1 | Service Water to Discharge Canal |