NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Millstone Nuclear Power Station Unit 3	0 5 0 0 0 4 2 3	1 OF 0 2

TITLE (4)

Feedwater Isolation Due to Main Steam/Feedwater Transient

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 2	0 6	8 6	8 6	–	0 1 2	– 0 0	0 3	0 7	8 6		0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)							
POWER LEVEL (10)	0 1 5		20.402(b)		20.405(c)	X	50.73(a)(2)(iv)		73.71(b)
			20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
			20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Paul G. Atkinson III, Associate Engineer	2 0 1 3 4 1 7 1 - 1 1 7 9 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
A	J I	P I T	W 3 5 1	N							

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	YES (If yes, complete EXPECTED SUBMISSION DATE)				
X	NO				

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

At 1025 hours on 2/6/86, while operating at 15% power, the plant received a Feedwater Isolation signal from the high-high water level off of Steam Generator "C". High water level in Steam Generator "C" was the end result of a Steam/Feedwater transient initiated by the Turbine Bypass (Steam Dump) valves modulating closed. All Engineered Safety Features Systems actuated properly. The restoration from Feedwater Isolation was performed in accordance with plant procedures.

Subsequent investigation revealed that the Main Steam Supply Header pressure signal was momentarily shorted out in the process of installing a temporary strip chart recorder. This caused the Main Steam Supply Header pressure to momentarily fail to zero (0) psig which caused the Turbine Bypass (Steam Dump) valves to modulate closed. The resulting Main Steam/Feedwater transient caused a high-high water level in Steam Generator "C" which initiated a feedwater isolation.

This report is being submitted in accordance with 10CFR50.73 (a) (2) (iv).

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Millstone Nuclear Power Station Unit 3	0 5 0 0 0 4 2 3	8 6	— 0 1 2	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1025 hours on 2/6/86, while operating at 15% power, the plant received a Feedwater Isolation signal from the high-high water level off of Steam Generator "C". High water level in Steam Generator "C" was the end result of a Steam/Feedwater transient initiated by the Turbine Bypass (Steam Dump) valves modulating closed. All Engineered Safety Features Systems actuated properly. The restoration from Feedwater Isolation was performed in accordance with plant procedures.

An investigation revealed that the Main Steam Supply Header pressure signal was momentarily shorted out during the installaton of a temporary strip chart recorder. The recorder was being installed to support Turbine Bypass (Steam Dump) valve testing as a part of the Power Ascension Test Program.

The Main Steam Supply Header pressure signal is provided as an analog signal for Turbine Bypass Valve control, Main Turbine Driven Feedpump speed control, and Main Control Board indication. Shorting out this signal caused Main Steam Supply header pressure to signal zero (0) psig. This resulted in the Turbine Bypass Valves modulating closed. The short was discovered immediately and removed. Upon removal of the short, the Turbine Bypass Valves modulated full open.

The Feedwater Bypass Flow Control Valves and Turbine Bypass Valves were changed to manual control to maintain Reactor Coolant System temperature and Steam Generator level. However, subsequent swelling in Steam Generator "C" initiated a high-high level Feedwater Isolation. All systems responded as required during the Feedwater Isolation and restoration was performed in accordance with plant procedures.

Further investigation revealed that the test probes on the leads assembled by the I&C department to connect the strip chart recorder to the Westinghouse 7300 Driver Card for the Main Steam Supply header pressure signal were of too great a length. Upon installation, the end of the pins shorted out on the Driver Card.

The I&C department modified the test probes on the leads used for connection of the strip chart recorder in this application and directed that all leads assembled to be used in similar applications be modified. In addition, a Technical Bulletin was issued within the I&C department and a department meeting was held to inform I&C technicians of this problem.

There were no safety implications to the public as all equipment performed its intended function.

This report is being submitted as required by 10CFR50.73 (a) (2) (iv).

NORTHEAST UTILITIES
THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

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March 7, 1986
MP-8793

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. NPF-49
Docket No. 50-423
Licensee Event Report 50-423/86-012-00

Gentlemen:

This letter forwards Licensee Event Report 86-012-00 required to be submitted within thirty days pursuant to 10CFR50.73 (a) (2) (iv), any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF).

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

WDR/PGA:se

Attachment: LER 86-012-00

cc: Dr. T. E. Murley, Region I

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