



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

September 21, 1988

MISSISSIPPI POWER & LIGHT COMPANY
SYSTEM ENERGY RESOURCES, INC.
SOUTH MISSISSIPPI ELECTRIC POWER ASSOCIATION
DOCKET NO. 50-416
GRAND GULF NUCLEAR STATION, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 47
License No. NPF-29

1. The Nuclear Regulatory Commission (the Commission) has found that
 - A. The application for amendment by System Energy Resources, Inc., (the licensee), dated July 15, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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- 2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-29 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 47, are hereby incorporated into this license. System Energy Resources, Inc. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- 3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by L. Kintner for

Elinor G. Adensam, Director
 Project Directorate II-1
 Division of Reactor Projects I/II
 Office of Nuclear Reactor Regulation

Attachment:
 Changes to the Technical
 Specifications

Date of Issuance: September 21, 1988

DFC	: LA: PD21: DRPR: PM: PD21: DRPR: LPEB: OLC	: OGC	: D: D2: DRPR	:
NAME	: PAnderson	: LKintner: ch: AGod	: EAdensam	:
DATE	: 9/19/88	: 9/19/88	: 9/20/88	:

ATTACHMENT TO LICENSE AMENDMENT NO. 47

FACILITY OPERATING LICENSE NO. NPF-29

DOCKET NO. 50-416

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Remove

6-7

Insert

6-7

ADMINISTRATIVE CONTROLS

PLANT SAFETY REVIEW COMMITTEE (PSRC) (Continued)

COMPOSITION

6.5.1.2 The PSRC shall be composed of the:

Chairman:	Manager, Plant Support
Vice Chairman/Member:	Manager, Plant Operations
Member:	Manager, Plant Maintenance
Member:	Operations Superintendent
Member:	Manager, Performance and System Engineering
Member:	Manager, Quality Services
Member:	Chemistry/Radiation Control Superintendent
Member:	I&C Superintendent
Member:	Plant Licensing Superintendent

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the GGNS General Manager to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSRC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or Vice Chairman.

QUORUM

6.5.1.5 The quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or Vice Chairman and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PSRC shall be responsible for review of:

- a. Station administrative procedures and changes thereto.
- b. The safety evaluations for (1) procedures, (2) changes to procedures, equipment or systems, and (3) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question and all programs required by Specification 6.8 and changes thereto.
- c. Proposed procedures and changes to procedures, equipment or systems which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed tests or experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- e. Proposed changes to Technical Specifications or the Operating License.