



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PUBLIC SERVICE COMPANY OF COLORADO

DOCKET NO. 50-267

FORT ST. VRAIN NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63  
License No. DPR-34

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Public Service Company of Colorado (the licensee) dated February 5, 1988 as supplemented June 23, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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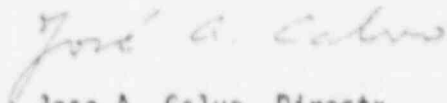
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. DPR-34 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jose A. Calvo, Director  
Project Directorate - IV  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 15, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 63  
TO FACILITY OPERATING LICENSE NO. DPR-34  
DOCKET NO. 50-267

Replace the following pages of the Appendix A Technical Specifications with the attached pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
7.1-7	7.1-7
7.1-8	7.1-8
7.1-9	7.1-9
7.1-10	7.1-10
7.1-13	7.1-13
7.1-22	7.1-22
7.1-23	7.1-23
7.4-1	7.4-1
7.4-2	7.4-2
7.4-3	7.4-3
7.5-3	7.5-3

Authorized deviations to the working hour guidelines shall be documented and available for Nuclear Regulatory Commission review.

3. Training

- a. A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10CFR Part 55. Compliance with Section 5.5 of ANSI 18.1-1971 shall be achieved no later than six months following commencement of commercial operation.
- b. A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Training Manager and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for Fire Brigade training/drill sessions which shall be held at least once per calendar quarter.
- c. The Technical Advisors shall have a bachelor degree or equivalent in a scientific or engineering discipline with specific training in plant design

and response and analysis of the plant for transients and accidents. An initial training and retraining program for the Technical Advisors shall be maintained under the direction of the Nuclear Training Manager. The Technical Advisors shall also receive training in plant design and layout, including the capabilities of instrumentation and controls in the Control Room.

Specification AC 7.1.2 - Plant Operations Review Committee (PORC), Administrative Controls

The organization, responsibilities, and authority of the PORC shall be as follows:

1. Membership

The Plant Operations Review Committee shall be composed of the following:

Chairman: Station Manager

Technical/Administrative Services Manager

Support Services Manager (Radiation Protection Manager)

Superintendent of Operations

Superintendent of Maintenance

Superintendent of Nuclear Betterment Engineering

| Results Engineering Supervisor  
| Administrative Shift Supervisor  
| Superintendent of Technical Services Engineering  
| Nuclear Training Manager

2. Alternates

| An alternate chairman and alternate members, if required, shall be appointed in writing by the PORC Chairman to serve in the absence of a chairman or a member; however, no more than two alternate members shall participate as voting members in PORC activities at any one time.

3. Meeting Frequency

| The PORC shall meet at least once per calendar month and as convened by the Chairman or his designated alternate.

4. Quorum

A quorum shall consist of the Chairman or alternate Chairman, and four members including alternates.

5. Responsibilities

The PORC shall be responsible for:

- 1
- a. Review of all procedures required by Technical Specification 7.4(a), (b), (c) and (d) and changes thereto, and any other proposed procedure or changes to approved procedures as determined by the Station Manager to affect nuclear safety.
  - b. Review of all proposed tests and experiments that affect nuclear safety.
  - c. Review of all proposed changes to the Technical Specifications.
  - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
  - e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering the evaluation and recommendations to prevent recurrence to the Manager, Nuclear Production and to the Chairman of the Nuclear Facility Safety Committee.
  - f. Review of all Reportable Events.
  - g. Review of facility operations to detect potential nuclear safety hazards.

- e) Instrumentation and Control
- f) Radiological Safety
- g) Mechanical and Electrical Engineering
- h) Quality Assurance Practices
- i) Other appropriate fields associated with the unique characteristics of the nuclear power plant.

## 2. Membership

The NFSC shall be composed of the following:

Chairman (As appointed - See Step 4)

Manager, Nuclear Licensing and Fuels Division

Manager, Nuclear Production Division

Manager, Nuclear Engineering Division

Manager, Quality Assurance Division

Safety and Security Director

| Consultants, as required, shall be appointed in writing by  
| the Vice President, Nuclear Operations.

## 3. Alternates

Alternate members, if required, shall be appointed in writing by the Vice President, Nuclear Operations; however, no more than two alternate members shall participate as voting members in NFSC activities at any one time.



**FORT ST. VRAIN NUCLEAR GENERATING STATION  
ADMINISTRATIVE AND DEPARTMENTAL MANAGEMENT ORGANIZATION CHART**

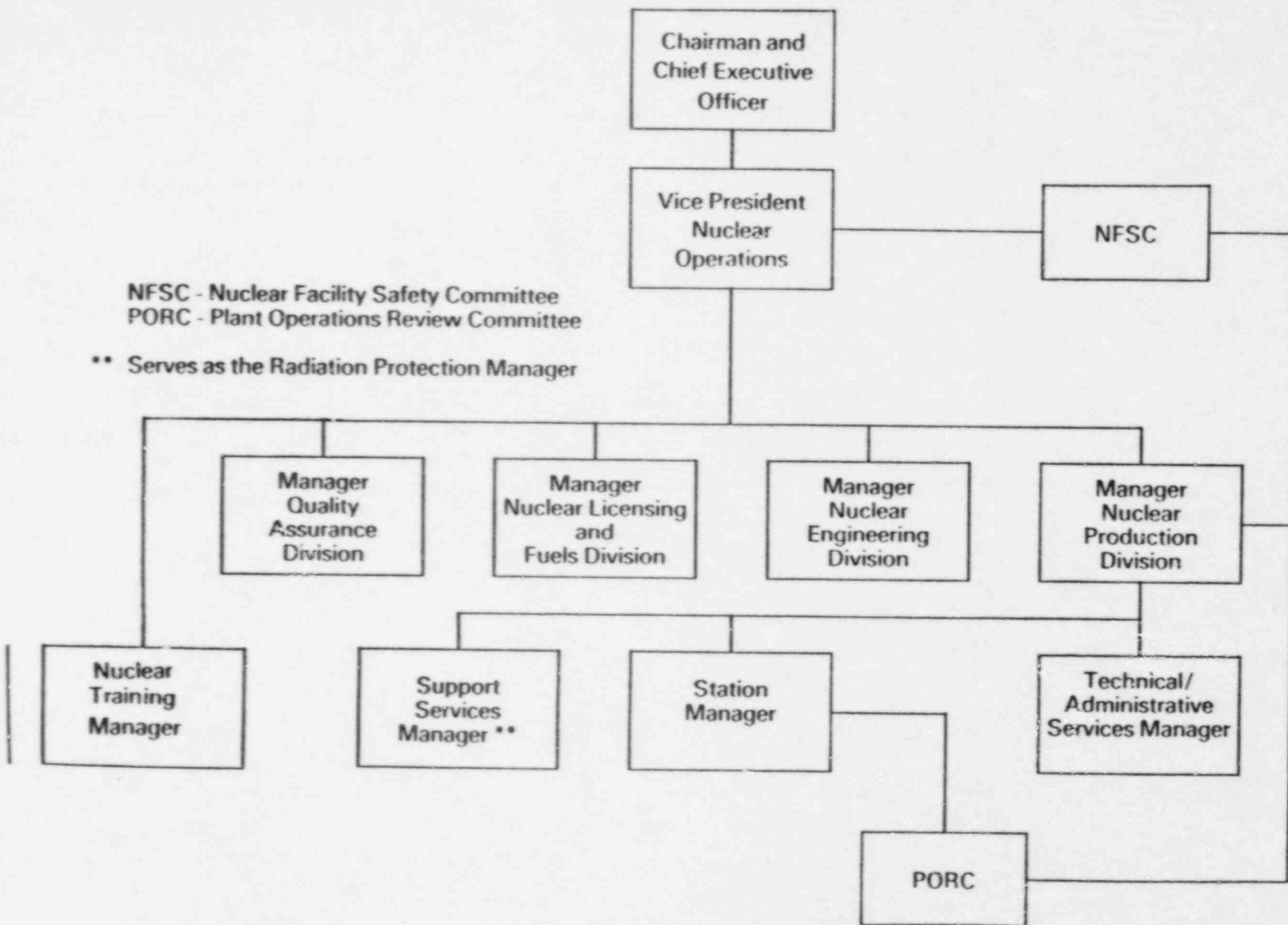
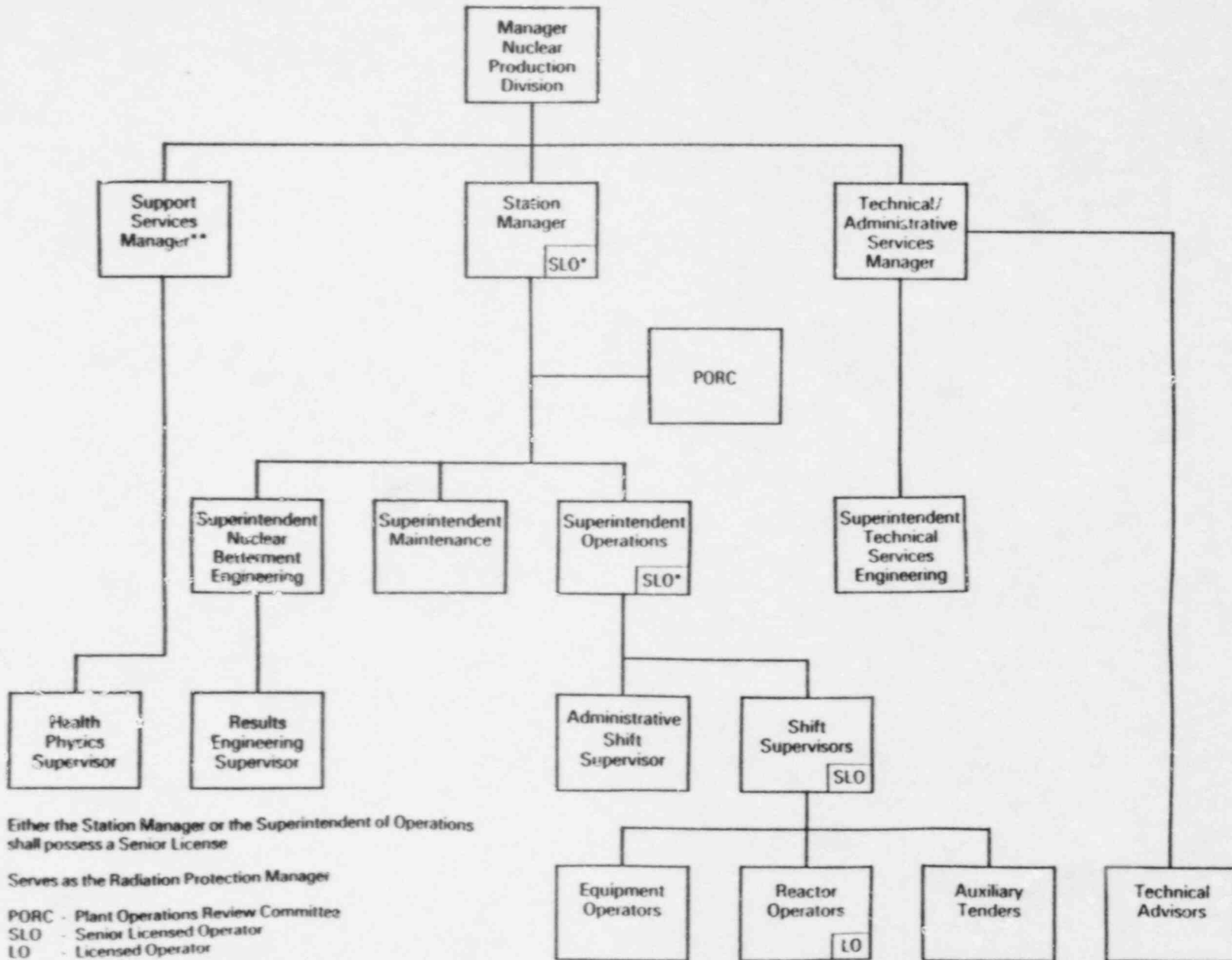


FIGURE 7.1 - 1

# FORT ST. VRAIN NUCLEAR GENERATING STATION CONDUCT OF OPERATIONS CHART



\* Either the Station Manager or the Superintendent of Operations shall possess a Senior License

\*\* Serves as the Radiation Protection Manager

PORC - Plant Operations Review Committee

SLO - Senior Licensed Operator

LO - Licensed Operator

Figure 7.1-2

7.4 PROCEDURES - ADMINISTRATIVE CONTROLS

Applicability

Applies to administrative procedures which will govern plant operations.

Objective

To ensure that written procedures will be maintained to define requirements for plant operation.

Specification AC 7.4 - Procedures, Administrative Controls

- a. Written procedures shall be established, implemented and maintained covering the activities referenced below:
1. The applicable procedures recommended in Appendix A of Safety Guide 33, November, 1972.
  2. Refueling operations.
  3. Surveillance and test activities of safety-related equipment.
  4. Security Plan implementation.
  5. Emergency Plan implementation.
  6. Process Control Program (PCP) implementation.

7. Offsite Dose Calculation Manual (ODCM) implementation.

8. Quality Assurance Program for effluent and environmental monitoring using the guidance in Regulatory Guide 1.21, Revision 1, June, 1974 and Regulatory Guide 4.1, Revision 1, April, 1975.

b. Procedures and administrative policies of a) above, and changes thereto, shall be reviewed by the PORC, or a subcommittee thereof, and approved by the appropriate management prior to implementation. Procedures shall be reviewed periodically as set forth in Administrative Procedures.

Security Plan procedures, and changes thereto, shall be reviewed by the Plant Operations Review Committee, or a subcommittee thereof, and approved by the designated Plant Security Officer prior to implementation.

c. Temporary changes to procedures of a) above may be made provided:

1. The intent of the original procedure is not altered.

2. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operators License.
  3. The change is documented, reviewed by the PORC, or a subcommittee thereof, and approved by the appropriate management within 14 days of implementation.
- d. Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20, and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

Respiratory protective equipment shall be provided in accordance with 10 CFR 20.103.

and surveillance, in-service inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totaling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the whole body dose received from external sources shall be assigned to specific major work functions.

c. Monthly Operating Report

A routine operating report covering the operation of the unit during the previous month shall be submitted prior to the fifteenth calendar day of the following month. Submittal shall be to the U. S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D. C. 20555, with a copy to the appropriate NRC Regional Administrator.