

October 9, 1998

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: **Docket Nos. 50-361 and 50-362
Corrected pages to Supplement 1 to Amendment Application Nos. 161 and
145, Appendix J, Primary Reactor Containment Leakage Testing
for Water-Cooled Power Reactors, Option B
San Onofre Nuclear Generating Station
Units 2 and 3**

- References:
1. Letter dated May 11, 1998, from Dwight E. Nunn (SCE) to Document Control Desk (NRC), Subject: Supplement 1 to Docket Nos. 50-361 and 50-362, Amendment Application Nos. 161 and 145, Appendix J, Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors, Option B, San Onofre Nuclear Generating Station, Units 2 and 3
 2. Letter dated May 30, 1996, from Dwight E. Nunn (SCE) to Document Control Desk (NRC), Subject: Docket Nos. 50-361 and 50-362, Amendment Application Nos. 161 and 145, Appendix J, Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors, Option B, San Onofre Nuclear Generating Station, Units 2 and 3

Enclosed are corrected pages to Supplement 1 to Amendment Application Numbers 161 and 145 to Facility Operating Licenses NPF-10 and NPF-15 for San Onofre Nuclear Generating Station (SONGS) Units 2 and 3, respectively. These amendment applications consist of Proposed Technical Specification Change Number NPF-10/15-468 (PCN 468), Supplement 1 submitted to the NRC by letter dated May 11, 1998 (Reference 1).

The corrected pages are required because 1) of changes to the Technical Specifications issued subsequent to the May 30, 1996, Submittal, Reference 2, and 2) to correctly identify P_a as the value of the calculated peak containment internal pressure related to the design basis loss-of-coolant accident (55.1 psig).

9810150209 981009
PDR ADOCK 05000361
P PDR

440159

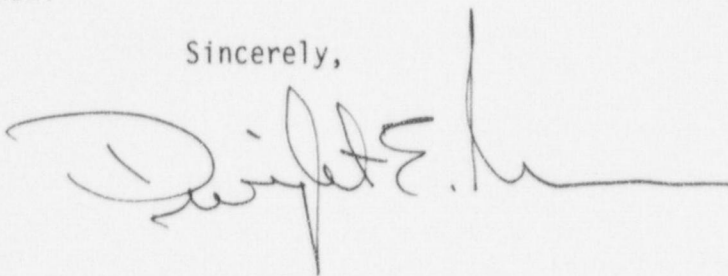
1/1
A017

Southern California Edison will continue to perform the containment leak rate testing required by the Technical Specifications using the calculated peak containment internal pressure related to the design basis main steam line break (56.6 psig) to be conservative. Therefore, the Significant Hazards Consideration is not affected by these corrections. Corrections are marked with change bars to show the differences from Supplement 1 to PCN 468.

In accordance with existing requirements, an Integrated Leak Rate Test (ILRT) will be required during the Units 2 and 3 Cycle 10 refueling outages. This ILRT would not be required by Option B of Appendix J. The Unit 2 Cycle 10 refueling outage is currently scheduled to begin on January 2, 1999. However, planning efforts for the ILRT need to begin as early as six months before the beginning of the outage. In addition, there are Local Leak Rate Tests (LLRTs) currently required to be performed beginning on November 18, 1998, which would not be required under Option B of Appendix J. These LLRTs would involve an unnecessary shutdown or a power entry into containment. Therefore, Southern California Edison requests approval of this proposed change no later than November 1, 1998, to be implemented within 30 days of issuance.

If you would like additional information on this Technical Specification change request, please let me know.

Sincerely,

A handwritten signature in cursive script, appearing to read "Dwight E. Hsu". The signature is written in dark ink and is positioned below the word "Sincerely,".

Enclosures

cc: E. W. Merschhoff, Regional Administrator, NRC Region IV
J. A. Sloan, NRC Senior Resident Inspector, San Onofre Units 2 & 3
J. W. Clifford, NRC Project Manager, San Onofre Units 2 and 3
S. Y. Hsu, Department of Health Services, Radiologic Health Branch