NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Millstone Nuclear Power Station Unit 3	0 5 0 0 0 4 2 3	1 OF 0 2

TITLE (4)

Feedwater Isolation Actuation - Excessive Feedwater Flow Due to Valve Leakage

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 2	0 4	8 6	8 6	— 0 0 9	— 0 0	0 3	0 4	8 6		0 5 0 0 0
										0 5 0 0 C

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 6	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Frances Sullivan, Associate Engineer	AREA CODE 2 0 3 4 4 7 - 1 7 9 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	J B	F C V	M 1 2 1	N							

SUPPLEMENTAL REPORT EXPECTED (14)

	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)				
X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 2/4/86 at 0026 hours, while operating at 6% power, a feedwater isolation (FWI) occurred due to high levels in Steam Generators 1 and 4. Plant Operators verified that all components required to function upon receipt of the FWI actuated properly. It was determined that the high steam generator levels were caused by excessive leakage through the motor-driven feedwater pump balancing valve (3FWS-PV590).

Subsequent investigation revealed that 3FWS-PV590 (an air-to-close valve) leaked due to a failure, caused by excessive vibration, of the copper tubing in the air actuator which prevented the valve from closing fully. In order to prevent a recurrence of this event, the air set has been moved to a non-vibrating mount adjacent to the valve and some of the copper tubing has been replaced with flexible hose.

This report is being submitted in accordance with 10CFR50.73 (a) (2) (iv).

IE22
1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Millstone Nuclear Power Station Unit 3	0 5 0 0 0 4 2 3	8 6	0 0 9	0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 2/4/86 at 0026 hours, while operating at 6% power, a feedwater isolation (FWI) occurred due to high levels in Steam Generators 1 and 4. Plant Operators verified that all components required to function upon receipt of the FWI actuated properly. It was determined that the high steam generator levels were caused by excessive leakage through the motor-driven feedwater pump balancing valve (3FWS-PV590).

Subsequent investigation verified that the cause of the high steam generator level was leakage through 3FWS-PV590, when the associated isolation valve was opened. Valve 3FWS-PV590 is an air-to-close valve, and the copper tubing to the "close" side of the diaphragm had failed due to a combination of vibration and excessive tightening of the compression fitting. In order to prevent a recurrence of this event, the valve air set has been moved to a non-vibrating mount adjacent to the valve. The tubing connections are now made of flexible armored hose and copper tubing with flexible loops to eliminate strain on the tubing connections.

There were no safety implications to the public since all components actuated to their correct positions upon receipt of the FWI.

This report is being submitted in accordance with 10CFR50.73 (a) (2) (iv).

NORTHEAST UTILITIES

THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

General Offices • Selden Street, Berlin, Connecticut

P.O. BOX 270
HARTFORD, CONNECTICUT 06141-0270
(203) 666-6911

March 4, 1986
MP-8773

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. NPF-49
Docket No. 50-423
Licensee Event Report 50-423/86-009-00

Gentlemen:

This letter forwards Licensee Event Report 86-009-00 required to be submitted within thirty days pursuant to 10CFR50.73 (a) (2) (iv), any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF).

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

WDR/FS:se

Attachment: LER 86-009-00

cc: Dr. T. E. Murley, Region I

IE22
1/1