PUBLIC SERVICE COMPANY OF COLORADO
FORT ST. VRAIN NUCLEAR GENERATING STATION

MONTHLY OPERATIONS REPORT
NO. 175
August, 1988

1/1

This report contains the highlights of the Fort St. Vrain, Unit No. 1, activities operated under the provisions of the Nuclear Regulatory Commission Operating License No. DPR-34. The report for the monthly partial scram/maximum temperature reports for control rod drive and orificing assemblies is not included this month because the reactor was not in operation. This report is for the month of August, 1988.

# 1.0 NARRATIVE SUMMARY OF OPERATING EXPERIENCE AND MAJOR SAFETY RELATED MAINTENANCE

The reactor remained shutdown the entire month of August, 1988, for helium circulator bolt replacement.

On August 16, 1988, Public Service Company of Colorado notified the NRC of a change in the CONTEMPT-G computer program utilized in developing the Fort St. Vrain E.Q. temperature profiles. This change caused the CONTEMPT-G program to calculate lower peak building temperatures for certain small line break scenarios. This event will be investigated and reported to the Nuclear Regulatory Commission in Licensee Event Report 88-012.

On August 29, 1988, at 1745 hours, while preparing "C" helium circulator for bolt replacement, five personnel received minor contamination on their hands and feet when helium and water sprayed out of the circulator interspace. The personnel and the area were decontaminated with soap and water. Calculated whole body doses were determined to be negligible.

During this month, bolts in Loop I helium circulators were replaced and the loop was returned to service. Bolts in Loop II helium circulators will be replaced during the month of September, 1988.

# 2.0 SINGLE RELEASES OF RADIOACTIVITY OR RADIATION EXPOSURE IN EXCESS OF 10% OF THE ALLOWABLE ANNUAL VALUE

None

3.0 INDICATION OF FAILED FUEL RESULTING FROM IRRADIATED FUEL EXAMINATIONS

None

4.0 MONTHLY OPERATING DATA REPORT

Attached

5.0 CONTROL ROD DRIVE PARTIAL SCRAM TEST RESULTS AND MAXIMUM DAILY TEMPERATURE REPORT

Not required this month.

OPERATING DATA REPORT

DATE September 15, 1988 COMPLETED BY M. L. Block TELEPHONE (303) 620-1180 OPERATING STATUS NOTES 1. Unit Name: Fort St. Vrain, Unit No. I 2. Reporting Period: 880801 through 880831 3. Licensed Thermal Power (MWt): 4. Nameplate Rating (Gross MWe): 342 5. Design Electrical Rating (Net Mwe): 330 6. Maximum Dependable Capacity (Gross MWe): 342 7. Maximum Dependable Capacity (Net MWe): 330 E. If Changes Occur In Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: None 9. Power Level To Which Restricted, If Any (Net Mwe): 270.6 10. Reasons For Restrictions, If Any: Reanalysis of safe shutdown cooling following a 90 minute interruption of forced cooling. This Month Year To Date Cumulative 11. Hours In Reporting Period 744 5,855 80,400 12 Number Of Hours Reactor Was Critical 0.0 3,798.1 37,244.8 13. Reactor Reserve Shutdown Hours 0.0 0.0 0.0 14. Hours Generator On-Line 0.0 25,072.8 3,487.3 15. Unit Reserve Shutdown Hours 0.0 0.0 0.0 16. Gross Thermal Energy Generated (MWK) 0.0 1,954,375.5 12,887,710.1 17. Gross Electrical Energy Generated (MWH) 0.0 718,184.0 4,260,404.0 18. Net Electrical Energy Generated (MWH) 3,761,071.0 -2188 670,805.0 19. Unit Service Factor 0.0 59.6 31.2 20. Unit Availability Factor 0.0 59.6 31.2 21. Unit Capacity Factor (Using MDC Net) 0.0 14.2 34.7 22. Unit Capacity Factor (Using DER Net) 0.0 34.7 14.2 23. Unit Forced Outage Rate 100.0 40.4 62.5 24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Helium Circulator Repairs, 880901, 960 hours. 25. If Shut Down At End Of Report Period, Estimated Date Of Startup: October 11, 1988 Forecast 26. Units In Test Status (Prior To Commercial Operation): Achieved INITIAL CRITICALITY N/A N/A INITIAL ELECTRICITY N/A NZA COMMERCIAL OPERATION N/A N/A

DOCKET NO.

50-267

### AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-267
Unit Fort St. Vrain Unit No. Date September 15,1988
Completed By M. L. Block Telephone (303) 620-1180

AUGUST Month

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0.0	17	0.0
2	0.0	18	0.0
3	0.0	19	0.0
4	0.0	20	0.0
5	0.0	21	0.0
6	0.0	22	0.0
7	0.0	23	0.0
8	0.0	24	0.0
9	0.0	25	0.0
10	0.0	26	0.0
1.7	0.0	27	0.0
12	0.0	28	0.0
.3	0.0	29	0.0
4	0.0	30	0.0
5	0.0	31	0.0
6	0.0		

<sup>\*</sup> Generator on line but no net generation.

ISP-3 Attachment TSP-3C Issue 2 Page 1 of 1

#### UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-267

UNIT NAME fort St. Vrain Unit No. 1

DATE September 15, 1988

COMPLETED BY M. L. Block

TELEPHONE [303] 620-1180

REPORT MONTH AUGUST, 1988

NO.	DATE	EVPE	LARATION	HEASON	[METHOD OF] [SHUTTING ] [DOWN ] [REACTOR ]	LER #	SYSTEM	COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
88-121	880801	1 5 1	794.0	24		N/A	AB	CMP	Reactor Manually Shutdown for Scheduled Replacement Of Helium Circulator Bolts. Refer To LER 87-019.
1 1 1									
1									
1									

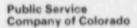
## REFUELING INFORMATION

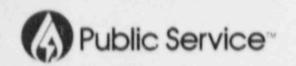
1.	Name of Facility	Fort St. Vrain Unit No. 2
2.	Scheduled date for next refueling shutdown.	April 4, 1989
3.	Scheduled date for restart following refueling.	May 27, 1989
4.	Will refueling or resumption of operation thereafter require a latechnical specification change or other license amendment?	No
	If answer is yes, what, in general, will these be?	***************************************
	If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10 CFR Section 50.59)?	No
	If no such review has taken   place, when is it scheduled?	1988
5.	Scheduled date(s) for submit- ting proposed licensing action   and supporting information.	
6.	Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	*******
7.	The number of fuel assemblies (a) in the core and (b) in the spent fuel storage rool.	a) 1482 HTGR fuel elements b) 0 spent fuel elements

### REFUELING INFORMATION (CONTINUED)

8.	The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one-third of core (approximately 500 HTGR elements). No change is planned.
9.	The projected date of the last   refueling that can be dis-   charged to the spent fuel pool   assuming the present licensed   capacity.	1996 under Agreements AT(04-3)-633 and DE-SU07-791D01370 between Public Service Company of Colorado, and General Atomic Company, and DOE.*

<sup>\*</sup> The 1996 estimated date is based on the understanding that spent fuel discharged during the term of the Agreements will be stored by DOE at the Idaho Chemical Processing Plant. The storage capacity has evidently been sized to accommodate eight fuel segments. It is estimated that the eighth fuel segment will be discharged in 1996.





16805 WCR 19 1/2, Platteville, Colorado 80651

September 15, 1988 Fort St. Vrain Unit No. 1 P-88336

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Docket No. 50-267

SUBJECT:

AUGUST 1988 MONTHLY

OPERATIONS REPORT

REFERENCE: Facility Operating

License Number DPR-34

Dear Sir:

Enclosed, please find the Monthly Operations Report for the month of August 1988, submitted per the requirements of Fort St. Vrain Technical Specification AC 7.5.1.

If you have any questions, please contact Mr. M. H. Holmes at (303) 480-6960.

Sincerely,

CHI, Faller bu

C. H. Fuller

Manager, Nuclear Production

Enclosure

cc: Regional Administrator, Region IV ATTN: Mr. T. F. Westerman, Chief Project Section B

> Mr. Robert Farrell Senior Resident Inspector, FSV Fort St. Vrain

CHF: kad