



Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401
Telephone (612) 330-5500

May 12, 1988

10 CFR Part 50
Section 50.62

Director of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-292 License Nos. DPR-42
50-306 DPR-60

Supplemental Information - Final AMSAC Design

Reference: (a) Letter dated April 22, 1988, from D M Musolf, NSP, to
Director NRR, NRC, "Description of Final AMSAC Design"

The purpose of this letter is to provide information to supplement
Reference (a). Documentation of this information was requested during
a telephone conference call with the NRC Staff on May 10, 1988.

The following additional information related to the final AMSAC design
selected for Prairie Island Units 1 and 2 is provided:

Separation Criteria

The separation requirements specified in Section 8.7 of the
Prairie Island Updated Safety Analysis Report (USAR) will be
complied with.

AMSAC Actuation Relays

Relays will be diverse, by manufacturer or by design, from relays
used in the Reactor Protection System (RPS).

Relays used as a safety-related circuit interface will meet class
1E requirements.

Relays used as a safety-related circuit interface will meet the
requirements of Appendix A of the July 7, 1986, NRC Safety
Evaluation Report (SER). Item (c) of this appendix is understood
to mean that the side of the device used in the safety-related
circuit shall be monitored while the fault is applied to the non-
safety related circuit side of the device.

A001
1/0

Director of NRR
May 12, 1988
Page 2

Northern States Power Company

The data showing qualification of applicable isolation devices to Appendix A of the July 7, 1986 SER will be available for a post-installation site audit of the AMSAC installation.

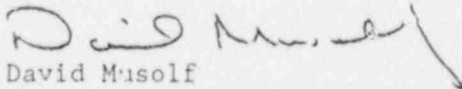
Human Factors Considerations

Any changes made to control room indication, controls, or alarms as a result of the AMSAC installation will have the approval of the Prairie Island Control Room Design Review Committee and conform to the standards described in the Detailed Control Room Design Review Report.

C20 Setpoint

The C20 setpoint will conform to WCAP 10858P-A Section 4.12.1, unless future concerns deem it appropriate to make this setpoint variable. NRC approval would be sought if a variable setpoint is used.

Please contact us if you have any questions related to the information we have provided.


David Misolf
Manager Nuclear Support Services

c: Regional Administrator, Region III, NRC
NRC Sr Resident Inspector
NRC Project Manager
G Charnoff