

APR 26 1988

MEMORANDUM FOR: Karl Kniel, Chief  
Reactor and Plant Safety Issues Branch  
Division of Reactor and Plant Systems, RES

FROM: Roy Woods, Senior Task Manager  
Reactor and Plant Safety Issues Branch  
Division of Reactor and Plant Systems, RES

SUBJECT: SUMMARY OF MARCH 31, 1988 MEETING TO DISCUSS REANALYSIS  
OF D&R RISK AT POINT BEACH, WITH FOLLOWUP CORRESPONDENCE

The meeting was held as described in the Meeting Notice (Attachment A).

In attendance were representatives from the NRC Staff, Sandia National Laboratory, NUMARC, EPRI, SAIC, Wisconsin Electric Power, Westinghouse, Southern Co. Services, CFA, Inc., and ERC International, as indicated on the attached Attendance sheet (Attachment B).

A package of material was distributed at the meeting (Attachment C) consisting of: (1) the subject EPRI/WOG/NUMARC analysis ("EPRI/WOG Analysis of Decay Heat Removal Risk at Point Beach," NSAC/113, March, 1988); (2) a package of slides that were presented at the meeting ("Differences between Sandia and NUMARC Analysis of Decay Heat Removal Related Risk for Point Beach," 31 March 1988, White Flint, Md.); (3) a cover letter with attachment to be discussed at the meeting (letter to Ms. Elaine Gorham-Bergeron, SNL, from Mr. John J. Haugh, NSAC, March 31, 1988, with attached ENCLOSURE, "Responses to SNL Questions on EPRI/WOG Analysis of DHR at Point Beach," March 30, 1988); (4) a set of three slides consisting of one map and two handwritten sheets regarding seismic hazards at various sites; and (5) a Sandia submittal for discussion at the meeting ("Supplemental Analyses and Comments/Responses to EPRI/WOG Analysis of Decay Heat Removal Risk at Point Beach," by David M. Ericson, Jr., ERC International for Sandia National Lab., Final Draft dated March 30, 1988).

The purpose of the meeting was to discuss differences between the subject industry-sponsored analysis of Point Beach, and an earlier analysis of the same plant which was sponsored by the NRC staff and performed by SNL as part of the resolution of Unresolved Safety Issue (USI) A-45, "Decay Heat Removal Requirements." The objective of the meeting was to provide the NRC Staff with sufficient information to allow completion of a preliminary staff evaluation of the industry-sponsored analysis. An earlier meeting had been held, during which NRC Staff representatives had outlined the subjects (to be discussed at this meeting) where more information was needed from industry representatives as bases for the industry sponsored study (Memorandum to Karl Kniel, NRC, from Roy Woods, NRC, "Minutes of Feb. 23, 1988 Meeting with Sandia and NUMARC Regarding EPRI/WOG/NUMARC Analysis of One USI A-45 Plant," March 3, 1988).

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Discussions concentrated on exploring core melt frequency differences, with a minor portion of the meeting devoted to cost and consequence differences as time permitted.

The package distributed at the meeting (Attachment C) presents the subjects discussed at this meeting in some detail. That level of detail will therefore not be re-iterated in this Summary.

In summary, it was considered reasonable by the NRC Staff and Sandia representatives to allow more credit for lower frequency of the SBLOCA, presence of the new batteries, and lack of dependence of the SI pumps upon availability of the CCW system (the SBLOCA frequency change is the dominant one) as proposed in the EPRI/WOG study. It was not considered prudent by NRC/SNL to allow more credit for many of the operator recovery actions proposed in the EPRI/WOG study.

The NRC staff believes that the approximate core damage frequency that would result from use of these agreements in a "revised" NRC staff sponsored analysis would be about  $9E-05$  per reactor year. This is below the NRC/SNL case study calculated value of  $3E-04$  per reactor year, and above the industry-sponsored study calculated result of  $1E-05$  per reactor year.

Additional details concerning differences between the two analyses are documented in Enclosure D. The material in this Enclosure was not presented at the meeting. Rather, it was produced as a result of the meeting and in fulfillment of the above stated objective of the meeting, to enable the NRC Staff to write a preliminary evaluation of the industry sponsored Point Beach Analysis. The material is presented with these minutes in order to provide a more complete, coherent record in one place for later reference. Enclosure D consists of the following: (1) Appendix D: Insights Gained From Industry-Sponsored Study of Point Beach, from the NRC Staff's Regulatory and Backfit Analysis: Unresolved Safety Issue A-45, Shutdown Decay Heat Removal Requirements, NUREG-1289 (Draft), April, 1988; (2) Letter from E. Bergeron (SNL) to R. Woods (NRC) April 27, 1988, with attached "Supplemental Analyses and Comments/Responses to EPRI/WOG Analysis of Decay Heat Removal Risk at Point Beach" by D. Ericson, Jr., ERC International, April 25, 1988.

Roy Woods, Senior Task Manager  
Reactor and Plant Safety Issues Branch  
Division of Reactor and Plant Systems, RES

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Meeting Attendees

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RWoods

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MCunningham

JMurphy

ARubin

FCoffman

OFC: RPSIB/DRPS \*

NAME: RWoods:sd

DATE: 4/ /88

RPSIB/DRPS \*

GMazetis

4/ /88

\*See previous concurrences

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Roy Woods, Senior Task Manager  
Reactor and Plant Safety Issues Branch  
Division of Reactor and Plant Systems, RES

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ARubin  
FCoffman

OFC: RPSIB/DRPS  
NAME: RWoods:sd  
DATE: 4/26/88

RPSIB/DRPS  
GMazetis  
4/27/88

Attachment BATTENDANCE

(3/31/88)

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T. Ryan (afternoon)	NRC/	(301) 492-3550
Carl Stoff	EPRI	