

MAY 13 1988

Docket No. 50-293

MEMORANDUM FOR: James T. Wiggins, Chief
Reactor Projects Branch No. 3

FROM: A. Randy Blough, Chief
Reactor Projects Section No. 3B

SUBJECT: PILGRIM STATUS REPORT FOR THE PERIOD APRIL 23 - MAY 6, 1988

Enclosed is the Pilgrim bi-weekly status report from the NRC Resident Office at Pilgrim. Three resident inspectors monitored activities at the plant during the report period. In addition, two region-based specialist inspectors were onsite to review the licensee's security management effectiveness and the licensee's program to control contaminated equipment during maintenance and testing at offsite facilities. NRC is conducting a public meeting at the Plymouth Memorial Hall, in Plymouth, Massachusetts on May 11, 1988 to discuss the evaluation of public comments and concerns expressed during the February 18, 1988 public meeting on the Boston Edison Company's Pilgrim Restart Plan. NRC Region I has issued a news release to inform the public of the scheduled meeting.

The status reports are intended to provide NRC management and the public with an overview of plant activities and NRC inspection activities. Subsequent inspection reports will address many of these topics in more detail.

original signed by:

A. Randy Blough, Chief
Reactor Projects Section No. 3B

Enclosure:
As stated

8805250076 880513
PDR ADDCK 05000293
R PDR

DEB
/

MAY 13 1988

cc w/encl:

R. Bird, Senior Vice President-Nuclear, BECo
L. Gustin, Vice President - Corporate Relations, BECo
K. Highfill, Station Director, BECo
J. Keyes, Regulatory Affairs Manager, BECo
Chairman, Board of Selectmen, Plymouth
Chairman, Board of Selectmen, Carver
Chairman, Board of Selectmen, Duxbury
Chairman, Board of Selectmen, Marshfield
Chairman, Board of Selectmen, Kingston
The Honorable Edward P. Kirby
The Honorable Peter Forman
The Honorable Edward J. Markey
M. Ernst, Committee on Energy, Commonwealth of Massachusetts
S. Pollard, Energy Secretary, Commonwealth of Massachusetts
B. McIntyre, Chairman, Department of Public Utilities
N. Johnson, Chairman, Duxbury Nuclear Committee
Plymouth Civil Defense Director
R. Boulay
M. Conyngham
M. Jeka
K. Anderson
Public Document Room (FDR)
Local Public Document Room (LPDR)
Nuclear Safety Information Center (NSIC)
NRC Resident Inspector
Commonwealth of Massachusetts (2)

MAY 13 1988

bcc w/encl:

Region I Docket Room (with concurrences)

T. Murley, NRR
F. Miraglia, NRR
T. Martin, NRR
S. Varga, NRR
B. Boger, NRR
D. Crutchfield, NRR
J. Partlow, NRR
C. Rossi, NRR
L. Shao, NRR
R. Wessman, NRR
D. McDonald, NRR
F. Akstulewicz, NRR
J. Roe, NRR
W. Russell, RI
W. Kane, RI
W. Johnston, RI
G. Sjoblom, RI
S. Collin, RI
R. Blough, RI
L. Doerflein, RI
M. Kohl, RI
M. J. DiDonato, RI (2)
K. Abraham, RI (2)

B

RI:DRP
CWarren/mjd

5/13/88

B

RI:DRP
RBlough

5/13/88



RI:DRP
JWiggins

5/13/88

ENCLOSURE

PILGRIM STATUS REPORT FOR THE PERIOD APRIL 23 - MAY 6, 1988

1.0 Plant Status

As of 8:00 a.m. on May 6, 1988, the reactor was in cold shutdown mode with moderator temperature about 93 degrees Fahrenheit.

2.0 Facility Operations Summary

The plant has been shutdown for maintenance and to make program improvements since April 12, 1986. The reactor core was completely defueled on February 13, 1987 to facilitate extensive maintenance and modification of plant equipment. The licensee completed fuel reload on October 14, 1987. Reinstallation of the reactor vessel internal components and the vessel head was followed by completion of the reactor vessel hydrostatic test. The primary containment integrated leak rate test was also completed during the week of December 21, 1987.

3.0 Items of Special Interest

NRC Public Meeting to Discuss Comments on BECo's Pilgrim Restart Plan

On May 11, 1988, NRC will be conducting a public meeting at the Plymouth Memorial Hall, in Plymouth, Massachusetts to respond to public comments and concerns on the Boston Edison Company's Pilgrim Restart Plan raised during the February 18, 1988 public meeting. The meeting is scheduled for 7:00 p.m. to 9:00 p.m. NRC Region I has issued a news release to inform the public of the scheduled meeting.

NRC Commissioner Rogers Tours Pilgrim

On May 6, 1988, NRC Commissioner Kenneth Rogers toured the licensee's Chiltonville Training facilities and the Pilgrim Station. The licensee presented a description of the new onsite organization, recent personnel changes, current plant status and the status of offsite emergency planning efforts. Handouts used by the licensee during the presentation will be attached to the next routine resident inspection report. Commissioner Rogers was accompanied by his staff, Mr. Samuel Collins of NRC Region I, and the resident inspectors.

NRC Assessment Panel Meeting

On April 26, 1988, members of NRC management from Region I met onsite and discussed inspection plans with the NRC resident staff. Members of NRR management participated in the meeting via teleconference. The NRC Assessment Panel meets periodically to coordinate the planning and execution of NRC inspection and licensing activities related to Pilgrim. The panel is chaired by Mr. Samuel J. Collins, Deputy Director, Division of Reactor Projects, NRC Region I.

Recent NRC Inspection Results

On April 22, 1988, NRC Region I issued the inspection report 50-293/88-02 documenting the results of a routine inspection conducted on January 25 - February 4, 1988 by a region based specialist inspector. The inspection focused on the licensee's Material Condition Improvement Action Program (MCIAP). Overall, the MCIAP was determined to be a viable program that will encompass programmatic improvements and it is being effectively implemented.

The inspection report 50-293/88-08 was issued on April 27, 1988 documenting the results of a special electrical team inspection conducted on February 1-5, 1988. Two violations were identified in the electrical maintenance area: 1) Inadequate maintenance of the 125V and 250V DC batteries due to the lack of acceptance criteria in the procedures resulted in battery terminal corrosion, gap between seismic support rails and the batteries, and the tightness of bolted connections on battery terminals. 2) Lack of surveillance testing of DC breakers. These findings indicate a general weakness in electrical maintenance. Three items remained unresolved at the end of the inspection pending licensee's corrective actions on: 1) the degraded voltage protection design deficiencies, 2) the undersized starter for motor control center breakers, and 3) the solid fuse links in the DC control circuits.

4.0 Emergency Notification System (ENS) Report

During this period, the licensee made the following reports to the NRC pursuant to 10 CFR 50.72:

- At approximately 2:40 p.m. on April 25, 1988, the "B" emergency diesel generator was started inadvertently due to a personnel error. The licensee I&C technicians were performing a calibration test on a pressure switch for the prelube pump on the "B" diesel. A technician inadvertently pushed the "local diesel start" button when he attempted to clear an alarm on the local panel. The "reset" button and the "local diesel start" button are on the same panel approximately 3 inches apart.

- On April 26, 1988 at about 3:22 a.m., the "B" reactor building closed cooling water (RBCCW) pump coupling failed during a normal operation. Low system pressure initiated auto-start of a standby "A" RBCCW pump in the same loop. There are two RBCCW loops and each loop has three pumps. A maintenance request has been initiated to repair the "B" RBCCW pump and the licensee's investigation is going. The licensee determined that the auto-start of the "A" pump could be considered as an Engineered Safety Feature actuation and subsequently notified the NRC via ENS at 11:41 a.m. on April 26, 1988.
- On April 26, 1988, at about 2:05 p.m., the control room received inadvertent automatic closure of the inboard containment isolation valves on the reactor water cleanup system (RWCU) suction and return lines. The licensee's preliminary investigation indicated that a type CR 120A electrical relay in the primary containment isolation system circuit had failed and de-energized the logic circuit for the RWCU isolation valves. The licensee initiated a maintenance request to replace the failed relay coil.
- On May 5, 1988, at about 11:00 a.m., the licensee experienced inadvertent Group II isolation including all outboard secondary containment dampers, an automatic start of the "B" standby gas treatment system, and an isolation of the outboard residual heat removal system to Radwaste primary containment isolation valve. The observed actuations occurred during a relay coil replacement in the Group VI reactor water cleanup (RWCU) system primary containment isolation logic circuit. The licensee's preliminary investigation determined that it was a personnel error. A licensee electrician failed to identify the correct logic circuit and mistakenly pulled a fuse from the Group II isolation logic circuit. Both Group II and Group VI isolation logic circuits are in C-942 panel in the cable spreading room. The fuse was reinserted and the actuation cleared a short time later.

The NRC inspector's review of these events will be further discussed in Inspection Report 50-293/88-19.

5.0 NRC Staff Status During the Period

The inspection staff at Pilgrim during the report period consisted of the following:

Clay Warren	---	Senior Resident Inspector
Jeffrey Lyash	---	Resident Inspector
Tae Kim	---	Resident Inspector

In addition, two region-based specialist inspectors were onsite during the report period to review: 1) the licensee's security management effectiveness (April 25 - May 6; IR 50-293/88-18) and 2) the licensee's program under materials license (No. 20-07626-02) to control contaminated equipment during maintenance and testing at offsite facilities (April 27 and 28; IR 30-10083/88-01). Also, during the weeks of April 25 and May 2, 1988, a five member inspection team from Region I were visiting the Pilgrim station and the licensee's Braintree engineering office to evaluate the licensee's maintenance program effectiveness (IR 50-293/88-17).