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Adaptation of OCA-P, a Probabilistic Fracture-Mechanics Code, to a Personal Computer

D. G. Ball R. D. Cheverton

Prepared for the U.S. Nuclear Regulatory Commission Office of Nuclear Regulatory Research Under Interagency Agreements DOE 40-551-75 and 40-552-75

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D. G. Ball* R. D. Cheverton

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NOTICE: This document contains information of a preliminary nature. It is subject to revision or correction and therefore does not represent a final report.

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FOREWORD

The work reported here was performed at Oak Ridge National Laboratory (ORNL) under the Heavy-Section Steel Technology (HSST) Program, C. E. Pugh, Program Manager. The program is sponsored by the Office of Nuclear Regulatory Research of the U.S. Nuclear Regulatory Commission

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- S. Yukawa, Evaluation of Periodic Proof Testing and Warm Prestressing Procedures for Nuclear Reactor Vessels, HSSTP-TR-1, General Electric Company, Schenectady, N. Y. (July 1, 1969).
- L. W. Loechel, The Effect of Testing Variables on the Transition Temperature in Steel, MCR-69-189, Martin Marietta Corporation, Denver, Colo. (November 20, 1969).
- P. N. Randall, Gross Strain Measure of Fracture Toughness of Steels, HSSTP-TR-3, TRW Systems Group, Redondo Beach, Calif. (November 1, 1969).
- C. Visser, S. E. Gabrielse, and W. VanBuren, A Two-Dimensional Elastic-Plastic Analysis of Fracture Test Specimens, WCAP-7368, Westinghouse Electric Corporation, PWR Systems Division, Pittsburgh, Pa. (October 1969).
- T. R. Mager and F. O. Thomas, Evaluation by Linear Elastic Fracture Mechanics of Radiation Damage to Pressure Vessel Steels, WCAP-7328 (Rev.), Westinghouse Electric Corporation, PWR Systems Division, Pittsburgh, Pa. (October 1969).
- W. O. Shabbits, W. H. Pryle, and E. T. Wessel, Heavy-Section Fracture Toughness Properties of A533 Grade B Class 1 Steel Plate and Submerged Arc Weldment, WCAP-7414, Westinghouse Electric Corporation, PWR Systems Divison, Pittsburgh, Pa. (December 1969).
- F. J. Loss, Dynamic Tear Test Investigations of the Fracture Toughness of Thick-Section Steel, NRL-7056, Naval Research Laboratory, Washington, D.C. (May 14, 1970).
- P. B. Crosley and E. J. Ripling, Crack Arrest Fracture Toughness of A533 Grade B Class I Pressure Vessel Steel, HSSTP-TR-8, Materials Research Laboratory, Inc., Glenwood, Ill. (March 1970).
- T. R. Mager, Post-Irradiation Testing of 2T Compact Tension Specimens, WCAP-7561, Westinghouse Electric Corporation, PWR Systems Division, Pittsburgh, Pa. (August 1970).
- T. R. Mager, Fracture Toughness Characterisation Study of A533, Grade B, Class 1 Steel, WCAP-7578, Westinghouse Electric Corporation, PWR Systems Division, Pittsburgh, Pa. (October 1970).

- T. R. Mager, Notch Preparation in Compact Tension Specimens, WCAP-7579, Westinghouse Electric Corporation, PWR Systems Division, Pittsburgh, Pa. (November 1970).
- N. Levy and P. V. Marcal, Three-Dimensional Elastic-Plastic Stress and Strain Analysis for Fracture Mechanics, Phase I: Simple Flawed Specimens, HSSTP-TR-12, Brown University, Providence, R.I. (December 1970).
- W. O. Shabbits, Dynamic Fracture Toughness Properties of Heavy Section A533 Grade B Class 1 Steel Plate, WCAP-7623, Westinghouse Electric Corporation, PWR Systems Division, Pittsburgh, Pa. (December 1970).
- P. N. Randall, Gross Strain Crack Tolerance of A533-B Steel, HSSTP-TR-14, TRW Systems Group, Redondo Beach, Calif. (May 1, 1971).
- H. T. Corten and R. H. Sailors, Relationship Between Material Fracture Toughness Using Fracture Mechanics and Transition Temperature Tests, T&AM Report 346, University of Illinois, Jrbana, Ill. (August 1, 1971).
- 16. T. R. Mager and V. J. McLoughlin, The Effect of an Environment of High Temperature Primary Grade Nuclear Reactor Water on the Fatigue Crack Growth Characteristics of A533 Grade B Class I Plate and Weldment Material, WCAP-7776, Westinghouse Electric Corporation, PWR Systems Division, Pittsburgh, Pa. (October 1971).
- N. Levy and P. V. Marcal, Three-Dimensional Elastic-Plastic Stress and Strain Analysis for Fracture Mechanics, Phase II: Improved Modelling, HSSTP-TR-17, Brown University, Providence, R.I. (November 1971).
- S. C. Grigory, Tests of 6-in.-Thick Flawed Tensile Specimens, First Technical Summary Report, Longitudinal Specimens Numbers 1 through 7, HSSTP-TR-18, Southwest Research Institute, San Antonio, Tex. (June 1972).
- P. N. Randall, Effects of Strain Gradients on the Gross Strain Crack Tolerance of A533-B Steel, HSSTP-TR-19, TRW Systems Group, Redondo Beach, Calif. (June 15, 1972).
- S. C. Grigory, Tests of 8-Inch-Thick Flawed Tensile Specimens, Second Technical Summary Report, Transverse Specimens Numbers 8 through 10, Welded Specimens Numbers 11 through 13, HSSTP-TR-20, Southwest Research Institute, San Antonio, Tex. (June 1972).
- L. A. James and J. A. Williams, Heavy Section Steel Technology Program Technical Report No. 21, The Effect of Temperature and Neutron Irradiation Upon the Fatigue-Crack Propagation Behavior of ASTM AS33 Grade B, Class 1 Steel, HEDL-TME 72-132, Hanford Engineering Development Laboratory, Richland, Wash. (September 1972).
- S. C. Grigory, Tests of 8-Inch-Thick Flawed Tensile Specimens, Third Technical Summary Report, Longitudinal Specimens Numbers 14 through 16, Unflawed Specimen Number 17, HSSTP-TR-22, Southwest Research Institute, San Antonio, Tex. (October 1972).

- S. C. Grigory, Tests of 6-Inch Thick Tensile Specimens, Fourth Technical Summary Report, Tests of 1-Inch-Thick Flawed Tensile Specimens for Size Effect Evaluation, HSSTP-TR-23, Southwest Research Institute, San Antonio, Tex. (June 1973).
- 24. S. P. Ying and S. C. Grigory, Tests of 6-Inch-Thick Tensile Specimens, Fifth Technical Summary Report, Acoustic Emission Monitoring of One-Inch and Six-Inch-Thick Tensile Specimens, HSSTP-TR-24, Southwest Research Institute, San Antonio, Tex. (November 1972).
- R. W. Derby, J. G. Merkle, G. C. Robinson, G. D. Whitman, and F. J. Witt, Test of 6-Inch-Thick Pressure Vessels. Series 1: Intermediate Test Vessels V-1 and V-2, ORNL-4895, Oak Ridge Natl. Lab., Oak Ridge, Tenn. (February 1974).
- W. J. Stelzman and R. G. Berggren, Radiation Strengthening and Embrittlement in Heavy Section Steel Plates and Welds, ORNL-4871, Oak Ridge Natl. Lab., Oak Ridge, Tenn. (June 1973).
- P. B. Crosley and E. J. Ripling, Crack Arrest in an Increasing K-Field, HSSTP-TR-27, Materials Research Laboratory, Inc., Glenwood, Ill. (January 1973).
- P. V. Marcal, P. M. Stuart, and R. S. Bettes, Elastic-Plastic Behavior of a Longitudinal Semi-Elliptic Crack in a Thick Pressure Vessel, HSSTP-TR-28, Brown University, Providence, R.1, (June 1973).
- W. J. Stelzman, R. G. Berggren, and T. N. Jones, ORNL Characterization of Heavy-Section Steel Technology Program Plates 01, 02 and 03, NUREG/CR-4092 (ORNL/TM-9491), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (April 1985).
- 30. Canceled.
- J. A. Williams, The Irradiation and Temperature Dependence of Tensile and Fracture Properties of ASTM A533, Grade B, Class 1 Steel Plate and Weldment, HEDL-TME 73-75, Hanford Engineering Development Laboratory, Richland, Wash. (August 1973).
- J. M. Steichen and J. A. Williams, High Strain Rate Teneile Properties of Irradiated ASTM A533 Grade B Class 1 Pressure Vessel Steel, Hanford Engineering Development Laboratory, Richland, Wash. (July 1973).
- P. C. Riccardella and J. L. Swedlow, A Combined Analytical-Experimental Fracture Study of the Two Leading Theorie of Elastic-Plastic Fracture (J-Integral and Equivalent Energy), WCAP-8224, Westinghouse Electric Corporation, Pittsburgh, Pa. (October 1973).
- R. J. Podlasek and R. J. Eiber, Final Report on Investigation of Mode III Crack Extension in Reactor Piping, Battelle Columbus Laboratories, Columbus, Ohio (December 14, 1973).
- 35. T. R. Mager, J. D. Landes, D. M. Moon, and V. J. McLaughlin, Interim Report on the Effect of Low Frequencies on the Fatigue Crack Growth Characteristics of A533 Grade B Class 1 Plate in an

Environment of High-Temperature Primary Grade Nuclear Reactor Water, WCAP-8256, Westinghouse Electric Corporation, Pittsburgh, Pa. (December 1973).

- 36. J. A. Williams, The Irradiated Fracture Toughness of ASTM A533, Grade B, Class 1 Steel Measured with a Four-Inch-Thick Compact Tension Specimen, HEDL-TME 75-10, Hanford Engineering Development Laboratory, Richland, Wash. (January 1975).
- 37. R. H. Bryan, J. G. Merkle, M. N. Raftenberg, G. C. Robinson, and J. E. Smith, Test of 6-Inch-Thick Pressure Vessels. Series 2: Intermediate Test Vessels V-3, V-4, and V-6, ORNL-5059, Oak Ridge Natl. Lab., Oak Ridge, Tenn. (November 1975).
- T. R. Mager, S. E. Yanichko, and L. R. Singer, Fracture Toughness Characterization of HSST Intermediate Pressure Vessel Material, WCAP-8456, Westinghouse Electric Corporation, Pittsburgh, Pa. (December 1974).
- 39. J. G. Merkle, G. D. Whitman, and R. H. Bryan, An Evaluation of the HSST Program Intermediate Pressure Vessel Tests in Terms of Light-Water-Reactor Pressure Vessel Safety, ORNL/TM-5090, Oak Ridge Natl. Lab., Oak Ridge, Tenn. (November 1975).
- 40. J. G. Merkle, G. C. Robinson, P. P. Holz, J. E. Smith, and R. H. Bryan, Test of 8-In.-Thick Pressure Vessels. Series 3: Intermediate Test Vessel V-7, ORNL/NUREG-1, Oak Ridge Natl. Lab., Oak Ridge, Tenn. (August 1976).
- J. A. Davidson, L. J. Ceschini, R. P. Shogan, and G. V. Rao, The Irradiated Dynamic Fracture Toughness of ASTM A533, Grade B, Class 1 Steel Plate and Submerged Arc Weldment, WCAP-8775, Westinghouse Electric Corporation, Pittsburgh, Pa. (October 1976).
- 42. R. D. Cheverton, Pressure Vessel Fracture Studies Pertaining to a PWR LOCA-ECC Thermal Shock: Experiments TSE-1 and TSE-2, ORNL/ NUREG/TM-31, Oak Ridge Natl. Lab., Oak Ridge, Tenn. (September 1976).
- 43. J. G. Merkle, G. C. Robinson, P. P. Holz, and J. E. Smith, Test of B-In.-Thick Pressure Vessels. Series 4: Intermediate Test Vessels V-5 and V-9 with Inside Nozzle Corner Cracks, ORNL/NUREG-7, Oak Ridge Natl. Lab., Oak Ridge, Tenn. (August 1977).
- J. A. Williams, The Ductile Fracture Toughness of Heavy Section Steel Plate, NUREG/CR-0859, Hanford Engineering Development Laboratory, Richland, Wash. (September 1979).
- 45. R. H. Bryan, T. M. Cate, P. P. Holz, T. A. King, J. G. Merkle, G. C. Robinson, G. C. Smith, J. E. Smith, and G. D. Whitman, Test of 8-in.-Thick Pressure Vessels. Series 3: Intermediate Test Vessel V-7A Under Sustained Loading, ORNL/NUREG-9, Oak Ridge Natl. Lab., Oak Ridge, Tenn. (February 1978).
- 46. R. D. Cheverton and S. E. Bolt, Pressure Vessel Fracture Studies Pertaining to a PWR LOCA-ECC Thermal Shock: Experiments TSE-3 and TSE-4 and Update of TSE-1 and TSE-2 Analysis, ORNL/NUREG-22, Oak Ridge Natl. Lab., Oak Ridge, Tenn. (December 1977).

- D. A. Canonico, Significance of Reheat Cracks to the Integrity of Pressure Vessels for Light-Water Reactors, ORNL/NUREG-15, Oak Ridge Natl. Lab., Oak Ridge, Tenn. (July 1977).
- 48. G. C. Smith and P. P. Holz, Repair Weld Induced Residual Stresses in Thick-Walled Steel Pressure Vessels, NUREG/CR-0093 (ORNL/NUREG/ TM-153), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (June 1978).
- P. P. Holz and S. W. Wismer, Half-Bead (Temper) Repair Welding for HSST Vessels, NUREG/CR-0113 (ORNL/NUREG/TM-177), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (June 1978).
- 50. G. C. Smith, P. P. Holz, and W. J. Stelzman, Crack Extension and Arrest Tests of Axially Flawed Steel Model Pressure Vessels, NUREG/ CR-0126 (ORNL/NUREG/TM-196), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (October 1978).
- R. H. Bryan, P. P. Holz, J. G. Merkle, G. C. Smith, J. E. Smith, and W. J. Stelzman, Test of 6-in.-Thick Pressure Vessels. Series 3: Intermediate Test Vessel V-7B, NUREG/CR-0309 (ORNL/NUREG-38), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (October 1978).
- R. D. Cheverton, S. K. Iskander, and S. E. Bolt, Applicability of LEFM to the Analysis of PWR Vessels Under LOCA-ECC Thermal Shock Conditions, NUREG/CR-0107 (ORNL/NUREG-40), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (October 1978).
- R. H. Bryan, D. A. Canonico, P. P. Holz, S. K. Iskander, J. G. Merkle, J. E. Smith, and W. J. Stelzman, Test of 6-in.-Thick Pressure Vessels, Series 3: Intermediate Test Vessel V-8, NUREG/CR-0675 (ORNL/NUREG-58), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (December 1979).
- R. D. Cheverton and S. K. Iskander, Application of Static and Dynamic Crack Arrest Theory to TSE-4, NUREG/CR-0767 (ORNL/NUREG-57), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (June 1979).
- J. A. Williams, Tensile Properties of Irradiated and Unirradiated Welds of A533 Steel Plate and A508 Forgings, NUREG/CR-1158 (ORNL/ Sub-79/50917/2), Hanford Engineering Development Laboratory, Richland, Wash. (July 1979).
- 56. K. W. Carlson and J. A. Williams, The Effect of Crack Length and Side Groovee on the Ductile Fracture Toughness Properties of ASTM A533 Steel, NUREG/CR-1171 (ORNL/Sub-79/50917/3), Hanford Engineering Development Laboratory, Richland, Wash. (October 1979).
- P. P. Holz, Flaw Preparations for HSST Program Vessel Fracture Mechanics Testing; Mechanical-Cyclic Pumping and Electron-Beam Weld-Hydrogen Charge Cracking Schemes, NUREG/CR-1274 (ORNL/ NUREG/TM-369), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (May 1980).
- S. K. Iskander, Two Finite Element Techniques for Computing Mode I Stress Intensity Factors in Two- or Three-Dimensional Problems, NUREG/CR-1499 (ORNL/NUREG/CSD/TM-14), Computer Sciences Div., Union Carbide Corp. Nuclear Div., Oak Ridge, Tenn. (February 1981).

- 59. P. B. Crosley and E. J. Ripling, Development of a Standard Test for Measuring K_{IG} with a Modified Compact Specimen, NUREG/CR-2294 (ORNL/Sub-817755/1), Materials Research Laboratory, Glenwood, 111. (August 1981).
- 60. S. N. Atluri, B. R. Bass, J. W. Bryson, and K. Kathiresan, NOZ-FLAW: A Finite Element Program for Direct Evaluation of Stress Intensity Factors for Pressure Vessel Nozzle-Corner Flaws, NUREG/CR-1843, (ORNL/NUREG/CSD/TM-18), Computer Sciences Div., Oak Ridge Gaseous Diffusion Plant, Oak Ridge, Tenn. (March 1981).
- A. Shukla, W. L. Fourney, and G. R. Irwin, Study of Emergy Loss and Its Mechanisms in Homalite 100 During Crack Propagation and Arrest, NUREG/CR-2150 (ORNL/Sub-7778/1), University of Maryland, College Park, Md. (August 1981).
- 62. S. K. Iskander, R. D. Cheverton, and D. G. Ball, OCA-I, A Code for Calculating the Behavior of Flaws on the Inner Surface of a Pressure Vessel Subjected to Temperature and Pressure Transients, NUREG/CR-2113 (ORNL/NUREG-84), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (August 1981).
- R. J. Sanford, R. Chona, W. L. Fourney, and G. R. Irwin, A Photoelastic Study of the Influence of Non-Singular Stresses in Fracture Test Specimens, NUREG/CR-2179 (ORNL/Sub-7778/2), University of Maryland, College Park, Md. (August 1981).
- 64. B. R. Bass, S. N. Atluri, J. W. Bryson, and K. Kathiresan, OR-FLAW: A Finite Element Program for Direct Evaluation of K-Factors for User-Defined Flaws in Plate, Cylinders, and Pressure-Vessel Nozzle Cormers, NUREG/CR-2494 (ORNL/CSD/TM-165), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (April 1982).
- B. R. Bass and J. W. Bryson, ORMGEN-3D: A Finite Element Mesh Generator for 3-Dimensional Crack Geometries, NUREG/CR-2997, Vol. 1 (ORNL/TM-8527/V1), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (December 1982).
- 66. B. R. Bass and J. W. Bryson, ORVIRT: A Finite Element Program for Energy Release Rate Calculations for 2-Dimensional and 3-Dimensional Crack Models, NUREG/CR-2997, Vol. 2 (ORNL/TM-8527/V2), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (February 1983).
- R. D. Cheverton, S. K. Iskander, and D. G. Ball, PWR Pressure Vessel Integrity During Overcooling Accidents: A Parametric Analysis, NUREG/CR-2895 (ORNL/TM-7931), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (February 1983).
- 68. D. G. Ball, R. D. Cheverton, J. B. Drake, and S. K. Iska der, OCA-II, A Code for Calculating Behavior of 2-D and 3-D Surface Flaws in a Pressure Vessel Subjected to Temperature and Pressure Transients, NUREG/CR-3491 (ORNL-5934), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (February 1984).
- 69. A. Sauter, R. D. Cheverton, and S. K. Iskander, Modification of OCA-I for Application to a Reactor Pressure Vessel with Cladding on

the Inner Surface, NUREG/CR-3155 (ORNL/TM-8649), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (May 1983).

- R. D. Cheverton and D. G. Ball, OCA-P, A Deterministic and Probabilistic Fracture-Mechanics Code for Application to Pressure Vessels, NUREG/CR-3618 (ORNL-5991), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (May 1984).
- 71. J. G. Merkle, An Examination of the Size Effects and Data Scatter Observed in Small Specimen Cleavage Fracture Toughness Testing, NUREG/CR-3672 (ORNL/TM-9088), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (April 1984).
- 72. C. E. Pugh et al., Heavy-Section Steel Technology Program Five-Year Plan FY 1983-1987, NUREG/CR-3595 (ORNL/TM-9008), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (April 1984).
- 73. D. G. Ball, B. R. Bass, J. W. Bryson, R. D. Cheverton, and J. B. Drake, Stress Intensity Factor Influence Coefficients for Surface Flaws in Pressure Vessels, NUREC/CR-3723 (ORNL/CSD/TM-216), Oak Ridge Natl. Lab., Oak Ridge, Tennessee (February 1985).
- 74. W. R. Corwin, R. G. Berggren, and R. K. Nanstad, Charpy Toughness and Tensile Properties of Neutron Irradiated Stainless Steel Submerged-Arc Weld Cladding Overlay, NUREG/CR-3927 (ORNL/TM-9309), Oak Ridge Natl. Lab., Oak Ridge, Tennessee (September 1984).
- 75. C. W. Schwartz, R. Chona, W. L. Fourney, and G. R. Irwin, SAMCR: A Two-Dimensional Dynamic Finite Element Code for the Stress Analysis of Moving CRacks, NUREG/CR-3891 (ORNL/Sub/79-7778/3), University of Maryland, College Park, MD (November 1984).
- 76. W. R. Corwin, G. C. Robinson, R. K. Nanstad, J. G. Merkle, R. G. Berggren, G. M. Goodwin, R. L. Swain, and T. D. Owings. Effects of Stainless Steel Weld Overlay Cladding on the Structural Integrity of Flawed Steel Plates in Bending, Series 1, NUREG/CR-4015 (ORNL/TM-9390), Oak Ridge Natl. Lab., Oak Ridge, Tennessee (April 1985).
- 77. R. H. Bryan, B. R. Bass, S. E. Bolt, J. W. Bryson, D. P. Edmonds, R. W. McCulloch, J. G. Merkle, R. K. Nanstad, G. C. Robinson, K. R. Thoms and G. D. Whitman, Pressurized-Thermal-Shock Test of 6-in.-Thick Pressure Vessels. PTSE-1: Investigation of Warm Prestressing and Upper-Shelf Arrest, NUREG/CR-4106 (ORNL-6135), Oak Ridge Natl. Lab., Oak Ridge, Tenn. (April 1985).
- 78. R. D. Cheverton, D. G. Ball, S. E. Bolt, S. K. Iskander and R. K. Nanstad, Pressure Vessel Fracture Studies Pertaining to the PWR Thermal-Shock Issue: Experiments TSE-5, TSE-5A and TSE-6, NUREG/CR-4249 (ORNL-6163), Martin Marietta Energy Systems, Inc., Oak Ridge Natl. Lab., Oak Ridge, Tenn. (June 1985).
- 79. R. D. Cheverton, D. G. Ball, S. E. Bolt, S. K. Iskander and R. K. Nanstad, Pressure Vessel Fracture Studies Pertaining to the PWR Thermal-Shock Issue: Experiment TSE-7, NUREG/CR-4304 (ORNL-6177), Martin Marietta Energy Systems, Inc., Oak Ridge Natl. Lab., Oak Ridge, Tennessee (August 1985).

- 80. R. H. Bryan, B. R. Bass, S. E. Bolt, J. W. Bryson, J. G. Merkle, R. K. Nanstad and G. C. Robinson, Test of 6-in.-Thick Pressure Vessels. Series 3: Intermediate Test Vessel V-8A - Tearing Behavior of Low Upper-Shelf Material, NUREG/CR-XXXX (ORNL-6187), Martin Marietta Energy Systems, Inc., Oak Ridge Natl. Lab., Oak Ridge, Tennessee (to be published).
- R. D. Cheverton and D. G. Ball, A Parametric Study of PWR Pressure Vessel Integrity During Overcooling Accidents, Considering Both 2-D and 3-D Flaws, NUREG/CR-4325 (ORNL/TM-9682), Martin Marietta Energy Systems, Inc., Oak Ridge Natl. Lab., Oak Ridge, Tennessee (August 1985).
- 82. E. C. Rodabaugh, Comments on the Leak-Before-Break Concept for Nuclear Power Plant Piping Systems, NUREG/CR-4305 (ORNL/Sub/62-22252/3), E. C. Rodabaugh Associates, Inc., Hilliard, Ohio (August 1985).
- 83. J. W. Bryson, ORVIRT.PC: A 2-D Finite Element Fracture Analysis Program for a Microcomputer, NUREG/CR-4367 (ORNL-6208), Martin Marietta Energy Systems, Inc., Oak Ridge Natl. Lab., Oak Ridge, Tennessee (October 1985).

ADAPTATION OF OCA-P, A PROBABILISTIC FRACTURE-MECHANICS CODE, TO A PERSONAL COMPUTER

D. G. Ball R. D. Cheverton

ABSTRACT

The OCA-P probabilistic fracture-mechanics code can now be executed on a personal computer with 512 kilobytes of memory, a math coprocessor, and a hard disk. A user's guide for the particular adaptation has been prepared, and additional importance sampling techniques for OCA-P have been developed that allow the sampling of only the tails of selected distributions. Features have also been added to OCA-P that permit RTNDT to be used as an "independent" variable in the calculation of P(F|E).

1. INTRODUCTION

OCA-P, which is a probabilistic fracture-mechanics code, was developed for the Nuclear Regulatory Commission (NRC) to help in the evaluation of pressurized water reactor (PWR) pressure-vessel integrity during pressurized thermal-shock (PTS) transients. The code was originally written for a main-frame computer, but because of the popularity of personal computers (PC), OCA-P has now been adapted to a PC. The adaptation requires 512 kilobytes of memory, a math coprocessor, and a hard disk. These requirements were met with an IBM PC-XT and PC-AT, using the IBM Professional 1.0 compiler.¹

This report provides the necessary user's guide information for running OCA-P on a PC. In addition, it describes improvements that have been made in the importance-sampling techniques that are used in OCA-F and discusses further the use of RTNDT as an independent variable in the probabilistic analysis. The original version of OCA-P is described in Ref. 2, and typical applications are found in Refs. 3 and 4.

2. USER'S GUIDE

OCA-P is composed of two parts: a one-dimensional heat transfer code (1-R) and a probabilistic fracture-mechanics code (PFM). A description of the input and output associated with OCA-P is given in Ref. 2. It should be noted that the plotting options are not yet available in the PC version.

The file names used by the PC version are given in Table 2.1. Although the digital output to each unit is routed to a specific file by the code, this may be overridden by using the DOS "SET" command. For example, during a run of the 1-R code, unit-1 output in a file named CASE1.BIN is obtained by issuing the commands:

SET ONERO1.BIN=CASE1.BIN ONER.

To obtain unit-6 output on the console during a run of the PFM code, issue the commands:

SET PFM.LPT=CON PFM.

Due to its size, the load module for the PFM portion of the code must be stored on a 1.2 megabyte floppy disk for transmittal, or, the source or object modules may be included on a 360 kilobyte floppy.

FORTRAN	File name	File type	Description
1-R Code:			
55 6 1	ONER.DAT ONER.LPT ONER01.BIN	Formatted Formatted Binary	Control input Digital output Data for input to PFM
PFM Code:			
55 56 1 2 7 8 6 17 29 19	PFM.DAT KICKIA.DAT ONERO1.BIN LI860945.BIN LI8603D.BIN CI860945.BIN PFM.LPT CCD.LPT PFMSUM.LPT Scratch	Formatted Formatted Binary Binary Binary Binary Formatted Formatted Formatted	Control input Override toughness curves Output from I-R code 2-D axial influence coefficients 3-D axial influence coefficients 2-D circ. influence coefficients Detailed digital output Deterministic summary Probabilistic summary

Table 2.1. OCA-P file information

Using the PROFORT linker, the PFM load module may be constructed from the object modules by the command:

LINK PFM1+PFM2, PFM, CON; .

After the input files are set up, the codes are run by typing ONER for the 1-R code and PFM for the PFM code. In addition, it is necessary to include the statement FILES=15 in the CONFIG.SYS file.

Input control data for a sample problem is contained in files ONER.DAT and PFM.DAT, which will be on the transmitted floppy disk. The problem is the same as that given in Appendix A of Ref. 2 with the exception that only one weld is analyzed in PFM using 10,000 trials. The execution times for the sample problem are given in Table 2.2 for different machines.

The PC version of OCA-P has a timing routine which writes the date and time of the run on unit 6. Also, the elapsed time of the run is calculated and included at the end of the unit-6 output.

Code	Computer	Execution time (minutes)	(PC time)/(3033 time)
I - R	PC PC-XT (hard disk) PC-AT IBM 3033	62.37 60.97 52.32 0.745	83.7 81.8 70.2
PFM	PC-XT (hard disk) PC-AT IBM 3033	18.98 16.98 0.307	61.8 '°.3

Table 2.2. Timing information on sample problem

3. IMPROVEMENTS TO OCA-P

Since the time that the OCA-P report was issued, several additions have been made to the code. This section discusses the additions and indicates the necessary corresponding changes in the user's manual. (These latter changes are included in detail in Appendix A.)

3.1 Inclusion of Improved Importance Sampling Capability

As discussed in Ref. 2, the OCA-P probabilistic model is based on Monte Carlo techniques; that is, a large number of vessels is generated, and each vessel is then subjected to a deterministic FM analysis to determine whether the vessel will fail. Each vessel is defined by randomly selected values of several parameters that are judged to have significant uncertainties associated with them. The calculated conditional probability of vessel failure [P(F|E)] is simply the number of vessels that fail divided by the total number of vessels generated. Thus,

$$P(F|E) = \sum_{j} \hat{P}_{j} v_{j} N_{j} \int_{0}^{W} f(a)B(a)da , \qquad (3.1)$$

where

 $P_{j} = \frac{N_{fj}}{N_{vj}^{2}},$

- N'_{fj} = number of vessels with a flaw in the jth region that fail,
- v_i = number of vessels simulated with a flaw in the jth region,
- V₄ = volume (or area) of jth region,
- Nj = flaws of all depths per unit volume (or surface area) of the jth region,
- a = flaw depth,
- f(a) = flaw-depth density function,
- B(a) = probability of nondetection,
 - w = wall thickness.

The parameters N and f(a) pertain to vessel conditions prior to preservice inspection and repair, and B(a) is derived on the basis of repairing or otherwise disposing of all detected flaws.

For very small values of P(F|E), the values of N^{*} required to achieve reasonable accuracy become quite large. Under some circumstances, the value of N^{*} can be reduced by using importance sampling techniques. This can be done in some cases by eliminating flaw depths that do not contribute significantly to initiation and by sampling only the tails of other distribution functions. The portion of the distribution function not sampled is accounted for by multiplying the number of simulated vessels, N^{*}_{vj}, by a correction factor. Equation (3.1) then becomes

$$P(F|E) = \sum_{j} \frac{\hat{P}_{j}}{\pi F_{kj}} N_{j} A_{j} \int_{0}^{W} f(a)B(a)da ,$$

where

Fki = correction factor for kth simulated parameter.

Values of F_k are a function of the points on the distribution curve at which sampling is started and stopped (truncation point). When importance sampling is used for the flaw-depth density function given in Ref. 2, and only the first flaw-depth increment is omitted,

$$F_k$$
 (flaw depth density) = $\frac{1}{\int_{\Delta a_1}^{W} f(a)B(a)da}$ = 3.24

Table 3.1 includes values of F_k for several different starting points on a normal distribution curve that is truncated at 3σ . As indicated, if the distribution is sampled above 1σ , F = 6; if it is sampled above 2σ , F = 46. If just two normal distributions are sampled, if they are sampled above 1.25σ , and if the first crack-depth increment can be omitted, $\Pi F_{kj} \approx 300$, which represents a significant savings in computer costs for the same accuracy in P(F|E). Of course, this type of importance sampling can only be used when the first crack-depth increment does not contribute much to P(F|E) and when P(F|E) is small enough

Identifying No. (NDLRS or NRTRS)	Start of sampling (number of stan- dard deviations above mean)	Fraction of distribution not simulated ²	F ₂ , F ₃
1	1.0	0.8422	6
2	1.25	0.8954	10
3	1 + 50	0,9343	15
4	1.75	0.9611	26
5	2.00	0.9784	46
6	2.25	0.9891	92
7	2.50	0.9951	204
8	2.75	0.9983	588

Table 3.1. Values of F_k for a normal distribution

"Assuming truncation at ±3a.

(3.2)

that only the tails of the distribution functions contribute significantly to P(F|E). (At the time of this writing, importance sampling is applied to the crack-depth density function and to the normal distribution for RTNDT and Δ RTNDT.)

3.2 The Use of RTNDT as an Independent Variable

In some cases it is convenient to use RTNDT as an independent variable in the probabilistic FM analysis, where RTNDT is the nil ductility reference temperature at the inner wirface. Values of RTNDT at any radial position in the wall can be estimated using Eq. (11) in Ref. 2 and the relation

where RTNDT₀ is the initial value of RTNDT, and Δ RTNDT is the increase in RTNDT due to radiation. However, since Δ RTNDT = f(F₀, Cu, Ni) [where F₀ is the fast-neutron fluence at the inner surface, and Cu and Ni are the concentrations of copper and nickel in the vessel material (wt Z)], RTNDT is not actually an independent variable; that is, the actual independent variables are RTNDT₀, F₀, Cu, and Ni. Thus, an error is introduced when RTNDT is used as the independent variable. This error is discussed below and in greater detail in Rei. 5.

A distribution function for $\Delta {\rm RTNDT}_{\rm S}$ can be obtained by performing a Monte Carlo analysis with

 $\Delta RTNDT$ (°C) = f(Cu, N1, F),

in which case F, Cu and Ni are simulated. However, as discussed in Ref. 5, the distribution is somewhat sensitive to the mean values of F_0 , Cu and Ni. Furthermore, for a given value of RTNDT, different combinations of RTNDT₀ and Δ RTNDT will result in somewhat different values of P(F|E). The size of the error depends on details of the transient and can be as large as an order of magnitude.⁵

References

- Professional FORTRAN, Version 1.0, IBM Personal Computer Software, by Ryan-McFarland Corporation, First Edition, November 1984.
- R. D. Cheverton and D. G. Ball, OCA-P, A Deterministic and Probabilistic Fracture-Mechanics Code for Application to Pressure Vessels, NUREG/CR-3618 (ORNL-5991), Union Carbide Corp., Nuclear Div., Oak Ridge Natl. Lab., Oak Ridge, Tenn. (May 1984).
- R. D. Cheverton, S. K. Islander and D. G. Ball, PWR Pressure Vessel Integrity During Overcooling Accidents: A Parametric Analysis, NUREG/CR-2895 (ORNL/TM-7931), Union Carbide Corp., Nuclear Div., Oak Ridge Natl. Lab., Oak Ridge, Tenn. (February 1983).
- ORNL PTS Study Group, Pressurized Thermal Shock Evaluation of the H. B. Robinson Unit 2 Nuclear Power Plant, NUREG/CR-4183 (ORNL/TM-9567), (in preparation).
- R. D. Cheverton, "Thermal-Shock Technology," Heavy-Section Steel Technology Program Semiannual Progress Report for October 1984-March 1985, NUREG/CR-4219, Vol. 1 (ORNL/TM-9593/V1), pp. 140-149, Martin Marietta Energy Systems, Inc., Oak Ridge Natl. Lab., Oak Ridge, Tenn.

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APPENDIX A. UPDATES TO ORIGINAL REPORT

Appendix A includes the revised probabilistic control input to the OCA-P code. The revised control input replaces pages 48-50 of Ref. 2. Also, a revised summary sheet for the probabilistic analysis (page 81 of Ref. 2) is included. There is a charge in the calculated probability of vessel failure that reflects the existence of an error in the code related to use of SI units in the Monte Carlo part of the analysis.

Card Type 6.0 - Probabilistic Control Parameters

0.11

Name	Variable	Default	Description
RDPTSS	NSIM	0	Maximum number of trials to be generated for each weld if IACCEL > 0, \neq 2: for each flaw depth if IACCEL < 0 or = 2.
			A positive value indicates use of fluence and copper on Card Type 6.2, whereas a negative value indicates the the use of Δ RTNDT on Card Type 6.3.
	IACCEL	0	Controls use of importance sampling on crack depth.
			 0 no importance sampling 1: first-increment flaw does not initiate but is counted in number of trials, thus accelerating conver- gence. 2: uniform sampling of crack depths (NSIM trials used for each crack
			 depth) 3: first and second-increment flaws do not initiate but are counted in number of trials thus accelerating convergence. < 0: only one crack depth is sampled; for instance, for IACCEL =-2 the second depth would be the only depth sampled (see Table 4.2 in Ref. 2).
	VOLWLO	1	Volume of weld 1 on Card Type 6.2, or 6.3, m ³ .
	WELDN	τ.	Number of flaws/m ³ .
	CONVPC	10	Maximum percent error in $P(F E)$ for each weld at a 95% confidence level.
	FLWSTR	551.7	Flow stress for plastic instability determination, MPa.
	USKIA	2.20	(K _{Ia}) _{max} , MPa+√m.
	NPCRK	9	Number of increments to be used for initial crack depth (maximum of 15).
	AWINT	4.3	Size of first crack-depth increment, mm.
	CDLIM	57.2	Extreme dimension of deepest crack-depth increment, mm.
			A geometric progression of NPCRK increments is generated between 0 and CDLIM to model initial crack depths. K ₁ values are sampled at midpoint of each increment.

Card Type 6.1 - Probabilistic Distribution Parameters

S/R Name	Variable	Default	Description
RDPTSS	SDFDRT	13	ARTNDT standard deviation, °C.
	SDFKIC	0.15	K _{Ic} standard deviation (fraction of mean). If positive, use ORNL mean curve, Eq. (3) (Ref. 2); if negative, use NRC mean curve, Eq. (5) (Ref. 2).
	SDFKIA	0.1	<pre>K_{Ia} standard deviation (fraction of mean). If positive, use ORNL mean curve, Eq. (4) (Ref. 2); if negative, use NRC mean curve, Eq. (6) (Ref. 2).</pre>
	SDLDRT	3.0	$\Delta RTNDT$ -standard truncation point (number of standard deviations).
	SDLKIC	3.0	K _{Ic} -distribution truncation point (number of standard deviations).
	SDLKIA	3.0	K _{Ia} -distribution truncation point (number of standard deviations).
	CONKIC	1.43	K _{Ic} -equation multiplier.
	CONKIA	1.25	K _{Ia} -equation multiplier.
	NDLRS	0	Controls use of importance sampling on $\overline{\Delta RTNDT}_{8}$:
			= 0: no importance sampling.
			> 0: sample only the tail of the distribution where NDLRS determines the starting point (see Table 3.1). Applies to all welds in Card Type 6.3.
	NRTRS	0	Controls use of importance sampling on RTNDT:
			= 0: no importance sampling.
			> 0: sample only the tail of the distribution where NRTRS determines the starting point (see Table 3.1). Applies to all welds in Card Type 6.3.

Card Type 6.2 - Weld Characteristics

Only required if NSIM > 0 (defined on Card Type 6.0).

Variable	Default	Description
NWELDS	1	Number of welds to be simulated, NWELDS \leq 6.
PFO(I)	t	Mean inside-surface fluence, neutrons/cm ² .
PCU(I)	1	Mean copper content, %.
PNI(I)	1	Mean nickel content, %.
PRTNO(I)	None	Mean initial reference temperature, °C.
SIGFO(I)	0.3	Fluence standard deviation, fraction of mean.
SIGCU(I)	0.025	Copper standard deviation, %.
SIGNI(I)	0.0	Nickel standard deviation, %.
SIGRTO(1)	9	Initial reference temperature standard deviation, °C.
VOLFAC(I)	1.0	Fraction to adjust $P(F E)_i$ for weld I based on the volume of the weld relative to weld 1. VOLFAC(1) = 1.0.
	Variable NWELDS PFO(I) PCU(I) PNI(I) PNI(I) SIGFO(I) SIGFO(I) SIGRI(I) SIGRI(I) SIGRTO(I)	VariableDefaultNWELDS1PFO(1)†PCU(1)†PCU(1)†PNI(1)†PRTNO(1)NoneSIGFO(1)0.3SIGCU(1)0.025SIGNI(1)0.0SIGRTO(1)9VOLFAC(1)1.0

The READ list is: NWELDS, [PFO(1), PCU(1), PNI(1), PRTNO(1), SIGFO(1), SIGCU(1), SIGNI(1), SIGRTO(1), VOLFAC(1), I = 1, NWELDS].

[†]If NWELDS = 1, these parameters are obtained from Card Type 4.0, if not provided here.

Card Type 6.3 - Weld Characteristics

Only required if NSIM < 0 (defined on Card Type 6.0).

S/R Name	Variable	Default	Description
ROPTSS	NWELDS	1	Number of welds to be simulated, NWELDS ≤ 6 .
	PFO(I)	None	ARTNDT
	PRTNO(I)	None	Mean initial reference, temperature, °C.
	SIGFO(I)	0.14	$\Delta RTNDT$ standard deviation, fraction of mean.
	SIGRTO(I)	9	Initial reference temperature standard deviation, °C.
	SIGNI(I)	None	Associated fluence (optional).
	VOLFAC(I)	1.0	Fraction to adjust $P(F E)_i$; for weld i based on the volume of the weld relative to weld 1. VOLFAC(1) = 1.0.

The READ list is: NWELDS, [PFO(1), PRTNO(1), SIGFO(1), SIGRTO(1), SIGNI(1), VOLFAC(1), I = 1, NWELDS].

TYPICAL P	OSTULATED I	RANSIEN	1. FLAWS/	M**3	F0 = 1.500D+19			
P(F/E)	UNADJUS 95%CI	SERR .	P(INITIA)	N*V	ADJUST P(F/E)	ED	NFAIL	MURIALS
1.60D-02 6.25D-03	7.64D-04 3.05D-04	4.79 4.88	1.60D-02 6.27D-03	1.000 0.500	1.60D-02 3.13D-03		1632 1597	60000 150000
			VE	SSEL	1.910-02	4.08		

EPI	HS FOR	INITIAL	INITL	ATION	(MM)					
		2.16	6.68	11.62	17.03	22.95	29.42	36.51	44.25	52.72
	NUMBER	37	2031	762	300	84	16	7	2	0
	PERCENT	1.1	62.7	23.5	9.3	2.6	0.5	0.2	0.1	0.0

EIGHTED	TIMES	OF	FAILURE	(MINUTE	S)							
		0.0	10.0	20.0	30.0	40.0	50.0	60.0	70.0	80.0	90.0	100.0
PERCEN	Т	0.	.0 0.	0 0.0	5.7	7 37.3	45.5	11.5	0.	0 0.0	0.	.0

INITIATION T-RINDT (DEG.C) -55.6 -41.7 -27.8 -13.9 0.0 13.9 27.8 41.7 55.6 69.4 83.3 97.2 111.1 NUMBER 0 14 239 1096 1432 442 18 0 0 0 0 PERCENT 0.0 0.4 7.4 33.8 44.2 13.6 0.6 0.0 0.0 0.0 0.0 0.0

ARREST T-RINDT (DEG.C) -27.8 -13.9 0.0 13.9 27.8 41.7 55.6 69.4 83.3 97.2 111.1 125.0 138.9 NUMBER 0 0 2 6 3 0 0 1 0 0 0 0 PERCENT 0.0 0.0 16.7 50.0 25.0 0.0 0.0 8.3 0.0 0.0 0.0 0.0

 KIC HISTOGRAM
 0.40
 0.50
 0.60
 0.70
 0.80
 0.90
 1.00
 1.10
 1.20
 1.30
 1.40
 1.50
 1.60

 NUMBER
 0
 85
 362
 845
 1011
 662
 241
 32
 3
 0
 0
 0

 PERCENT
 0.0
 2.6
 11.2
 26.1
 31.2
 20.4
 7.4
 1.0
 0.1
 0.0
 0.0

ICRMTP = 1 IACCEL = 0 NDLRS = 0 NRTRS = 0 DATE: 10/03/85 TIME: 22.21.17

CPU TIME: 4 MIN 21 SEC.

WELD

1

2

NUREG/CR-4468 ORNL/CSD/TM-233 Dist. Category RF

Internal Distribution

1-5.	D.	G.	Ball 25	. J. G. Merkle
6.	В.	R.	Bass 26	. A. P. Malinauskas
7.	s.	Ε.	Bolt 27	. R. K. Nanstad
8.	R.	н.	Bryan 28	. D. J. Naus
9.	J.	W.	Byson 29-33	. C. E. Pugh
10-14.	R.	D.	Cheverton 34	. G. C. Robinson
15.	J.	м.	Corum 35	. H. E. Trammell
16.	W.	Ε.	Corwin 36-40	. G. D. Whitman
17.	J.	s.	Crowell 41	. ORNL Patent Office
18.	D.	М.	Eissenberg 42	. Central Research Library
19.	G.	Ε.	Giles 43	. Document Reference Section
20-23.	s.	К.	Iskander 44-45	. Laboratory Records Department
24.	J.	J.	McGowan 46	· Laboratory Records (RC)

External Distribution

- C. Z. Serpan, Division of Engineering Technology, Nuclear Regulatory Commission, Washington, DC 20555
- M. Vagins, Division of Engineering Technology, Nuclear Regulatory Commission, Washington, DC 20555
- Director, Division of Reactor Safety Research, Nuclear Regulatory Commission, Washington, DC 20555
- Office of Assistant Manager for Energy Research and Development, Department of Energy, Oak Ridge Operations Office, Oak Ridge, TN 37831
- 51-52. Technical Information Center, DOE, Oak Ridge, TN 37831
- 53-327. Given distribution as shown in Category RF (NTIS-10)

BIBLIOGRAPHIC DATA SHEET	NUREG/CR-4468 ORNL/CSD/TM-233	
Adaptation of OCA-P, A Probabilistic Fracture- Mechanics Code, to a Personal Computer 5 Authonis D. G. Ball* and R. D. Theverton		
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